

**22.351 Systems Analysis of the Nuclear Fuel Cycle  
Fall 2005**

**Lab #4: MCNP PWR Pin Cell Model**

A typical PWR unit cell MCNP model is given on the course web (file PIN.inp). It models the geometry of representative PWR lattice cell with 4.5 w/o enriched fuel. Typical parameters are:

Fuel (UO <sub>2</sub> ) Enrichment	4.5 w/o
Fuel (UO <sub>2</sub> ) Density	10.4 g/cm <sup>3</sup>
Lattice Pin Pitch	1.26 cm
Fuel Temperature	900 K
Pellet Radius	0.4096 cm
Gap Thickness	0.0082 cm
Rod Diameter	0.9500 cm
Water Temperature	583.1 K
System Pressure	15.5 MPa
Power Density	104.5 kW/liter-core

- (a) Using the given MCNP model, run MCNP and calculate the following reaction rates (tally F4)
- U-235 fission rate (use FM = -6)
  - U-238 capture rate (use FM = 102)

Furthermore, assume the two group model takes the boundary of 0.625 eV, and calculate the epithermal and thermal reactions for each reaction. Compute spectrum indices based on these reaction rates:

$$C^* = \frac{\text{U-238 captures}}{\text{U-235 fissions}},$$

$$\delta_{25} = \frac{\text{epithermal U-235 fissions}}{\text{thermal U-235 fissions}},$$

$$\rho_{28} = \frac{\text{epithermal U-238 captures}}{\text{thermal U-238 captures}}.$$

- (b) Calculate and plot the neutron spectrum inside the fuel pellet, i.e., use a detailed energy structure as provided. Since it is straightforward to see that a harder spectrum can be achieved by either a higher reload fuel enrichment ( $X$ ) or a smaller hydrogen-to-heavy-metal ( $H/HM$ ) ratio, one can naturally give an asymptotic dependence of the epithermal-to-thermal flux ratio on the above two variables as

$$\frac{\phi_2}{\phi_1} \approx \frac{H/HM}{X}$$

Explain the physical meaning of this equation.

- (c) Modify the input file by adding a tally to obtain thermal,  $\phi_2$ , and epithermal flux,  $\phi_1$ , in the moderator. Calculate ratio of  $\phi_2/\phi_1$  in the fuel and moderator. Discuss relative magnitude of the two and the reasons.