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Pfirsch-Schlüter Electric Field in a Tokamak

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Pfirsch-Schlüter Electric Field in a Tokamak

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Abstract

A concise and complete differential equation determining the Pfirsch-Schlüter radial electric field in an up-down symmetric tokamak is presented in the limit of weak poloidal magnetic field.

The differential equation for the Pfirsch-Schlüter electric field in a tokamak has been evaluated for large aspect ratio circular flux surfaces [1], for general cross sections [2,3], and for tokamaks having the poloidal magnetic field B_p small compared to the toroidal magnetic field B_t with a weak ion diamagnetic drift [4]. Recent measurements in Alcator C-Mod [5] and DIII-D [6] for subsonic ion flows indicate that the ion diamagnetic flow must be retained since it tends to cancel with the function, ζ is the toroidal angle variable, $I = RB_t$, R is the major radius, and $\overline{}$ E × \overline{a} B drift velocity, where \overline{a} $B = I(\psi)\nabla \zeta + \nabla \zeta \times \nabla \psi$ is the axisymmetric tokamak magnetic field with ψ the flux \overline{a} \overline{a} E = $-(\partial \Phi/\partial \psi)\nabla \psi$ is the lowest order electric field with the electrostatic potential Φ a flux function to lowest order. Consequently, it is convenient to use the results of [2-4] to generalize the $B_p \ll B_t$ expression of [4] so that it retains the ion diamagnetic drift. To do so is very straightforward since the result of [4] is consistent with the $B_p \ll B_t$ limit of [2,3] within the ion diamagnetic drift term that now has to be retained. Defining the ion density, temperature and pressure as n, T and p, and taking $Z = 1$ for the ion charge number, the lowest order ion flow velocity \overline{a} V on a flux surface is the usual Pfirsch-Schlüter result of Hazeltine [7]:

$$
\vec{V} = \omega R^2 \nabla \zeta + u(\psi) \vec{B}
$$
 (1)

with

$$
\omega = -c[(\partial \Phi/\partial \psi) + (en)^{-1}(\partial p/\partial \psi)],\tag{2}
$$

and

$$
u = -\frac{cI}{e} \left[\frac{1.78}{\langle B^2 \rangle} + \frac{0.057 \langle (\nabla_{\parallel} \ell n B)^2 \rangle}{\langle (\nabla_{\parallel} B)^2 \rangle} \right] \frac{\partial T}{\partial \psi} , \qquad (3)
$$

 \overline{a} where $B =$ \overline{a} \mathbf{B} l, $\nabla_{\parallel} = \mathbf{B}^{-1}$ $\overline{}$ $\mathbf{B} \cdot \nabla$, and $\langle \ldots \rangle$ denotes a flux surface average. The departure of ∂Φ/∂ψ from a generalized radial Maxwell-Boltzmann relation (ω = constant) for an updown symmetric tokamak is then found from the radial flux of toroidal angular momentum by employing

$$
-\frac{5}{6pv}\langle R^2\nabla\zeta\cdot\vec{\pi}\cdot\nabla\psi\rangle = \left\langle \frac{R^4B_p^2}{\Omega^2} \right\rangle \frac{\partial\omega}{\partial\psi} + 0.19 \frac{cI^2T_e(\partial T/\partial\psi)^2}{eT(T+T_e)\langle\Omega^2\rangle} \left\langle R^2 \left(1 - \frac{B^2}{\langle B^2\rangle}\right) \right\rangle
$$

-0.53 $\frac{cT^{1/2}}{en^2} \frac{\partial}{\partial\psi} \left\{ \frac{I^2n^2\langle B^2\rangle}{T^{1/2}\langle\Omega^2\rangle} \frac{\partial T}{\partial\psi} \left[\langle R^2\rangle \left\langle \left(\frac{1}{B} - \frac{B}{\langle B^2\rangle}\right)^2 \right\rangle \right] - \left\langle \frac{R^2}{B^2} \left(1 - \frac{\langle R^2\rangle}{R^2}\right) \right\rangle \right\} ,$ (4)

where T_e is the electron temperature, $v = (4\pi^{1/2}e^4nln\Lambda)/3M^{1/2}T^{3/2}$ is the ion-ion collision frequency, and \overline{a} $\ddot{\pi}$ is the ion stress tensor. Setting

$$
\langle R^2 \nabla \zeta \cdot \vec{\pi} \cdot \nabla \psi \rangle = 0 \tag{5}
$$

 field once a boundary condition is specified. Except for the retention of the ∂p/∂ψ term in gives the differential equation determining the steady state Pfirsch-Schlüter radial electric the ω appearing in (4), Eqs. (2) and (4) are the same as Eqs. (68) and (75) of Ref. [4] for an up-down symmetric tokamak. Moreover, Eqs. (2) and (4) correspond precisely to the $B_p \ll B_t$ limit of Eq. (60) of Ref. [2], and are consistent with the T_e = T large aspect ratio result given in Ref. [1] for which the terms in the second line of Eq. (4) may be neglected as smaller than those in the first line by an inverse aspect ratio squared. The up-down asymmetric term of Refs. [2,4] may be added into (4) as necessary.

The correct isothermal limit is now recovered [8] by (5), while for the more typical non-isothermal cases the ∂p/∂ψ term in ω is comparable to the small numerical coefficient second term on the right when $T\partial[n^{-1}\partial p/\partial \psi] \sim 0.2q^2(\partial T/\partial \psi)^2$, with q the safety factor. In fact, the ion pressure gradient term is required to understand the Alcator C-Mod measurements in collisional to semi-collisional pedstals [5] because it nearly cancels the ∂Φ/∂ψ term in ω. Moreover, based on the entropy production argument of [9], in a banana regime pedestal of poloidal ion gyroradius width, T∂p/∂ψ >> p∂T/∂ψ is required, as has been observed in DIII-D [6].

The temporal evolution of the electrostatic potential is found from the flux surface average of conservation of toroidal angular momentum:

$$
\frac{\partial}{\partial t} \left[Mn(\omega \langle R^2 \rangle + Iu) \right] + \frac{1}{V'} \frac{\partial}{\partial \psi} \left[V' \langle R^2 \nabla \zeta \cdot \vec{\pi} \cdot \nabla \psi \rangle \right] = 0 \quad , \tag{6}
$$

where $V' = \langle ... \rangle$.

 differential equation determining the Pfirsch-Schlüter radial electric field in an up-down In summary, we have presented a concise and complete expression for the symmetric tokamak in the limit of weak poloidal magnetic field.

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