# ENGINEERING OF NUCLEAR REACTORS

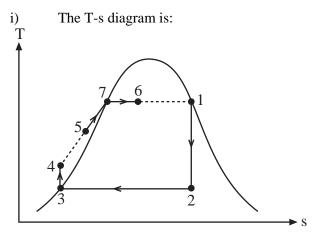
Thursday, October 14<sup>th</sup>, 2004, 9:30 – 11:00 a.m.

# **OPEN BOOK**

# **QUIZ #1 SOLUTIONS**

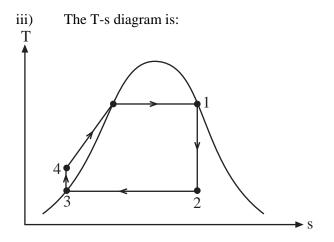
1.5 HOURS

### **Problem 1 (45%)**



ii) Turbine inlet (**Point 1**):  $T_1=280^{\circ}$ C,  $P_1=64$  bar,  $h_1=2780$  kJ/kg,  $s_1=5.9$  kJ/kg·K,  $x_1=1.0$ Turbine outlet (**Point 2**):  $T_2=30^{\circ}$ C,  $P_2=0.04$  bar,  $s_2=s_1=5.9$  kJ/kg·K,  $x_2=(s_2-s_{f2})/(s_{g2}-s_{f2}) \approx 0.69$ ,  $h_2=h_{f2}+x_2(h_{g2}-h_{f2}) \approx 1797$  kJ/kg Condenser outlet (**Point 3**):  $T_3=30^{\circ}$ C,  $P_3=0.04$  bar,  $h_3=126$  kJ/kg,  $x_3=0$ Pump outlet (**Point 4**):  $T_4 \approx 30^{\circ}$ C,  $P_4=64$  bar,  $h_4=h_3+(P_4-P_3)v_{f4} \approx 132.4$  kJ/kg Recirculation line (**Point 7**):  $T_7=280^{\circ}$ C,  $P_7=64$  bar,  $h_7=1236$  kJ/kg,  $x_7=0$ Core inlet (**Point 5**):  $P_5=64$  bar,  $h_5=0.1 \cdot h_4+0.9 \cdot h_7 \approx 1126$  kJ/kg Core outlet (**Point 6**):  $T_6=280^{\circ}$ C,  $P_6=64$  bar,  $h_6=h_{f6}+0.1 \cdot (h_{g6}-h_{f6}) \approx 1390$  kJ/kg

Thermal efficiency =  $(W_{turb}-W_{pump})/Q_{in} = [0.1(h_1-h_2)-0.1(h_4-h_3)]/(h_6-h_5) \approx 37\%$ or, equivalently, =1-Q<sub>out</sub>/Q<sub>in</sub> = 1-0.1(h\_2-h\_3)/(h\_6-h\_5) \approx 37\%



The thermal efficiency =  $(W_{turb}-W_{pump})/Q_{in} = [(h_1-h_2)-(h_4-h_3)]/(h_1-h_4) \approx 37\%$ , i.e., identical to the cycle with recirculation. This is expected because recirculation does not change either the net work done by the cycle or the external heat input to the cycle.

- iv) Advantages of using cycle without recirculation:
  - No steam separator, no recirculation line, thus lower capital cost

Disadvantages of using cycle without recirculation:

- Lower water density in the core, thus worse moderation
  - Large temperature rise in the core, bad for thermal stresses (will learn more on this subject later in the course)
  - Worse heat transfer (will learn more on this subject later in the course)

#### **Problem 2 (55%)**

i) The amount of He initially in the primary system,  $N_1$ , is readily obtained from the equation of state:

$$N_1 = \frac{P_{1i}V_1}{RT_{1i}} \approx 250,000 \text{ mol}$$
 (1)

where  $P_{1i}$  (=7 MPa) and  $T_{1i}$  (=673 K) are the initial pressure and temperature in the primary system, respectively, and  $V_1$  (=200 m<sup>3</sup>) is the primary system volume. Similarly, the amount of He initially in the containment,  $N_2$ , is found as:

$$N_2 = \frac{P_{2i}V_2}{RT_{2i}}$$
(2)

Where  $P_{2i}$  (=0.1 MPa) and  $T_{2i}$  (=300 K) are the initial pressure and temperature in the containment, respectively. However, the containment volume,  $V_2$ , is unknown. When the large break LOCA occurs, the gas inventories in the primary system and containment mix; thus the final pressure,  $P_f$  (=1.3 MPa), can be related to the gas inventories, the containment volume and the final (unknown) temperature,  $T_f$ , as:

$$P_{f} = \frac{(N_{1} + N_{2})RT_{f}}{V_{1} + V_{2}}$$
(3)

Equations 2 and 3 have three unknown ( $T_f$ ,  $V_2$  and  $N_2$ ), so a third equation is needed to solve the problem. The conservation of energy for the control volume representing the primary system and containment is:

$$\mathbf{U}_{\mathrm{f}} - \mathbf{U}_{\mathrm{i}} = \mathbf{0} \tag{4}$$

(note that the decay heat addition is negligible because we are to assume that instantaneous equilibrium is achieved). With reference to the two gas inventories, Equation 4 can be rewritten as:

$$(U_{1f} - U_{1i}) + (U_{2f} - U_{2i}) = 0 \quad \text{or} \quad N_1 c_v (T_f - T_{1i}) = N_2 c_v (T_{2i} - T_f) \quad (5)$$

Where  $c_v$  is the helium specific heat. Then  $T_f$  is readily obtained as:

$$T_{f} = \frac{N_{1}T_{1i} + N_{2}T_{2i}}{N_{1} + N_{2}}$$
(6)

Substituting Equation 2 and 6 in Equation 3, and solving for V<sub>2</sub>, the following result is obtained:

$$V_{2} = \frac{RN_{1}T_{1i} - P_{f}V_{1}}{P_{f} - P_{2i}} \approx 950 \text{ m}^{3}$$
(7)

And back-substituting in Equation 2 and 6,  $N_2$  ( $\approx$ 38,100 mol) and  $T_f$  ( $\approx$ 623 K) can be found.

ii) Indicating with  $\dot{Q}_{o}$  (=300 MW) the nominal reactor power, the pressure (and temperature) in the containment will continue to rise until the emergency cooling system capacity (=0.02 $\dot{Q}_{o}$ ) matches the decay heat rate (=0.06 $\dot{Q}_{o}$ t<sup>-0.2</sup>). Thus, solving for t, one obtains:

$$t_{\text{peak}} = \left(\frac{0.06}{0.02}\right)^{1/0.2} \approx 243 \text{ s}$$
 (8)

where  $t_{peak}$  is the time at which pressure and temperature peak. The net heat input to the containment, Q, between t=0 and t= $t_{peak}$  can be calculated as:

$$Q = \int_{0}^{t_{\text{peak}}} (0.06 \dot{Q}_{0} t^{-0.2} - 0.02 \dot{Q}_{0}) dt \approx 364.5 \text{ MJ}$$
(9)

The energy equation yields:

$$(N_1 + N_2)c_v(T_{peak} - T_f) = Q$$
(10)

And the peak temperature is:

$$T_{peak} = T_f + \frac{Q}{(N_1 + N_2)c_v} \approx 724 \text{ K}$$
 (11)

So the peak pressure is:

$$P_{f} = \frac{(N_{1} + N_{2})RT_{peak}}{V_{1} + V_{2}} \approx 1.5 \text{ MPa}$$
(12)

- iii) Advantages:
  - Lower loads on the containment.

- Can reduce containment thickness, which results in lower capital costs.

Disadvantages:

- Release of potentially radioactive gas to the environment, depending on the efficiency of the filter.
- If the vent valve failed open, the containment would lose its function.