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# **ANALYSIS OF MIXING DATA RELEVANT TO WIRE WRAPPED FUEL ASSEMBLY THERMAL-HYDRAULIC DESIGN**

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ANALYSIS OF MIXING **DATA** RELEVANT TO WIRE-WRAPPED **FUEL** ASSEMBLY THERMAL-HYDRAULIC DESIGN

**by**

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#### ABSTRACT

In this report analysis of recent experimental data is presented using the ENERGY code. **A** comparison of the accuracy of three types of experiments is also presented along with a discussion of uncertainties in utilizing this data for various code calibration purposes. The existence of internal swirl is discussed. The two empirical coefficients in ENERGY are determined from the data within a certain range of accuracy. This range is dictated to a large extent **by** the accuracy of the experiments and to a smaller extent **by** the ability of the code to utilize all sets of data in each experiment.

The effect of geometry and bundle size on mixing and swirl flow is discussed. **A** realistic estimate of the degree of accuracy within which we can predict temperature distribution within the bundle and along the duct of a 217-pin wire wrapped fuel assembly of an LMFBR is presented.

Gaps in data which need to be filled in to enhance our confidence in predicting coolant temperature distributions in a 217-pin LMFBR fuel bundle, are given. **A** brief description of two experiments that would fill these data gaps is presented. **A** novel experiment which

would be very useful for both fuel and poison assembly mixing studies is described. Conclusions drawn from this study are believed to be quite general in nature.

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#### **1.** INTRODUCTION

In the design of a wire wrapped Liquid Metal Fast Breeder Reactor (LMFBR) fuel assembly,coolant mixing caused by wire wraps is the most significant mode of energy redistribution<sup>(1,2)\*</sup>. At the high Reynolds number range of steady state operation of a typical LMFBR energy transport **by** conduction plays a secondary role only to energy transport **by** wire wrap mixing.

The maximum burnup (Mwd/T) for current LMFBR designs is very sensitive to the peak coolant temperature in the hot channel<sup>3</sup>, which in turn could depend, to a large extent, on interchannel mixing fates. In addition, the fuel assembly housing bowing is also of special concern. Bowing of the hexagonal housing is caused **by** differential thermal expansion. It is greatly enhanced **by** radiation induced stainless steel swelling in a fast spectrum and the temperature dependence of this induced swelling. Thus realistic predictions of assembly temperature distribution could have important effects on the flow housing design considerations. At present a purely theoretical analysis is not adequate to establish the extent of mixing caused **by** wire wraps in LMFBR assemblies. In order to reduce the uncertainty in predicting mixing rates caused **by** wire wraps (also known as flow sweeping **-** see Section **7.2** and Ref. 2 for a detailed description of various other modes

**\*** superscripts refer to references cited

of energy transport) many different types of experiments have been performed. Some of these experiments, in addition to providing data from which mixing-coefficients can be determined for use in computer programs, provide useful information on flow and pressure distribution in the rod bundle.

The three types of experiments that are most frequently used for determining wire-wrap mixing in a rod bundle can be classified according to their boundary conditions.

**1)** Continuous injection of an electrolyte salt solution at a point in the bundle. Electrical conductivity measurements at various points in the rod bundle can give an estimate of the amount of salt transported **by** mixing. From this a mixing coefficient can be derived.

2) Continuous local injection of heated coolant. Temperature measurement in the three dimensional bundle matrix will yield a mixing coefficient with appropriate data analysis. **3)** Employment of one, several, or a complete array of heated pins.<sup>4</sup> By measuring temperature distributions in the bundle, one can derive a mixing coefficient.

The mixing coefficient one would like to derive from these experiments should solely represent the contribution of wire-wraps in promoting energy transport from one point to another within the bundle. However, as indicated in Ref. 2,

other modes of energy transport co-exist (e.g. diversion cross-flow, flow scattering, energy transfer **by** turbulence) although their magnitudes may be considerably smaller than the wire wrap sweeping effect. It is often not possible to separate (except for thermal conduction) the individual contributions of the mixing mechanisms, from these experiments, to a good degree of accuracy. This is one of the reasons why computer programs like  $ENERGY$ <sup>1</sup> combine all the mixing mechanisms into one lumped mixing coefficient which is empirically determined. Other computer programs make an effort to determine the individual contributions of the wire-wrap, sweeping, diversion cross-flow and turbulence energy exchange and linearly add them. Linear addition of these effects is questionable. However, since the flow sweeping **by** wire wraps so completely dominates the mixing process, errors in either neglecting or linearly adding the effect of other mixing mechanisms are expected to be small. In addition to the experiments mentioned before, experimenters have used several techniques to measure axial and circumferential velocities in wire wrapped rod bundles. One of these is the use of Laser Doppler Velocimeter to determine velocity and turbulent intensity.

In this respect analysis of recent experimental data is presented using the ENERGY code. **A** comparison of the accuracy of three types of experiments is also presented along with a discussion of uncertainties in utilizing this data for

various code calibration purposes. The existence of internal swirl is discussed. The two empirical coefficients in ENERGY are determined from the data within a certain range of accuracy. This range is dictated to a large extent **by** the accuracy of the experiments and to a smaller extent **by** the ability of the code to utilize all sets of data in each experiment. The effect of geometry and bundle size on mixing and.swirl flow is discussed. **A** realistic estimate of the degree of accuracy within which we can predict temperature distribution within the bundle and along the duct of a 217-pin wire wrapped fuel assembly of an LMFBR is presented. Gaps in data, which need to be filled in, to enhance our confidence in predicting coolant temperature distributions in a 217-pin LMFBR fuel bundle, are given. **A** brief description of two experiments that would fill these data gaps is presented. **A** novel idea for an experiment which would be very useful for both fuel and poison assemblies mixing studies is described.

It is necessary to point out that the conclusions drawn from this study regarding flow distributions within the fuel assembly, internal swirl, bundle size effects, effect of geometry on mixing and swirl are quite general in nature. The code used to obtain these conclusions is the ENERGY code. However, the results and conclusions are expected to be independent of the tool used to obtain them.

#### 2. Theoretical Background

The  $ENERGY^1$  computer program models the periodic sweep flow through rod gaps as an enhanced effective eddy diffusivity. The model subdivides the fuel assembly into two predominant regions. In the central region the enhanced effective eddy diffusivity is responsible for energy transport in the transverse directions. Near the duct wall, flow is unidirectional and parallel to the duct wall. Energy transport can occur both **by** diffusion and **by** convection, although it has been found that the eneray transport **by** convection (swirl flow) predominates. ENERGY models the heat generation in the fuel rods as a volumetric heat source in the fluid. Until more data is available ENERGY uses two options for flow split. **1)** Hydraulic diameter flow split, 2) uniform velocity in the entire bundle. Each set of data was analyzed **by** taking two splits into consideration. The available data yields some useful information on this flow split, as is shown later.

The ENERGY code requires as input two empirical constants  $\overline{\epsilon_{\text{H}}^*}$  and  $\text{C}_1$ . All the mixing effects (e.g. turbulent exchange, diversion cross-flow, flow sweep due to wire wraps) are lumped into  $\overline{\epsilon_{\rm H}^*}$  =  $\overline{\epsilon_{\rm H}}$ /(Vde)]. It should however be noted that the effect of wire wrap for the current fuel assembly design almost completely dominates other effects included in  $\overline{\epsilon_{\rm H}^{\star}}$ . C<sub>1</sub> represents the swirl flow in the wall region and is defined as the ratio of the velocity in the gap between rod

and wall to the average axial bundle velocity  $(C_1 = \overline{V}g/\overline{V})$ . It is necessary to note that  $C_1$  is assumed to be uniform around the bundle and represents the average circumferential swirl velocity in the gap between rod and wall. The three types of experiments, namely, electrolyte salt injection, hot water injection experiment, and heated pins, are used to determine  $\overline{\epsilon_{H}^{*}}$  and  $C_{1}$ . Based on physical insight and inspection of the fundamental energy balance equation we can determine that  $\overline{c_{\rm H}^*}$  and  $C_1$  are dependent on the following parameters,

$$
\overline{\epsilon_{\mathrm{H}}^*} = f(h/d, p/d, \mathrm{Re}, \mathrm{Pr})
$$
 (1)

$$
C_1 = f(h/d, p/d, Re, Pr)
$$
 (2)

Since these experiments have been performed for fuel assemblies of different geometries and at various Reynolds numbers it is possible to determine these functional relationships for  $\overline{\epsilon_{\text{H}}^*}$ and  $C_1$ .

The heated pin experiments provide a boundary condition which represents the actual LMFBR fuel assembly assembly heat transfer boundary condition. The parameters  $\overline{\epsilon_{H}^{*}}$  and  $C_1$  in ENERGY are varied until coolant temperatures predicted **by** the code match the experimentally measured temperatures.It was found that  $\overline{\epsilon_{\text{H}}^*}$  and  $\overline{C_1}$  can be found independently by matching the central region and wall region temperature maps respectively. Apparently, swirl flow effects, fortunately, do not appear to penetrate significantly within the fuel assembly. Thus  $\overrightarrow{e_H}$  can

be determined from the data in the inner region and C<sub>1</sub> from the data on the wall region.

In order to show that both the salt injection and hot water injection experiments would also yield a mixing coefficient which can directly be used in the ENERGY code for LMFBR fuel assembly temperature calculations, it is necessary to realize that  $\overline{\epsilon_{\text{H}}^*}$  predominantly represents the wire sweep flow mixing effect. Thus the mixing coefficients determined from equations **(3)** and (4) below need not be modified (as is normally done in literature when mass eddy diffusivities are converted to heat eddy diffusivities or vice versa) in any manner for use in ENERGY.

The equations governing salt diffusion in salt injection experiments and heat diffusion in hot water injection experiments along with injection boundary conditions are given below (note: we may only concern ourselves with the steady state equation),

 $\frac{\text{Hot Water}}{\text{Mater}}$   $\frac{\text{DT*}}{\text{at}} = \frac{1}{\text{at}} \left( \frac{\text{H}}{\text{at}} + \frac{1}{\text{at}} \right) \sqrt{2} \text{T*}$  (3) Injection<sup>:</sup> Dt  $\overline{P}$  Re  $\overline{Y}$  Pr  $T - T_{COT.D}$ when  $z=0$ ,  $T^* = 1$  where  $T^* = \frac{C}{T}$  $HOT$ <sup>+</sup> COLD

Water is generally used as the working fluid. One finds  $\frac{\Pi}{\gamma}$  >>  $\frac{1}{\text{Pr}}$ 

$$
\frac{\text{salt}}{\text{Injection}}: \quad \frac{\text{DC*}}{\text{Dt}} = \frac{1}{\text{Re}} \left( \frac{\epsilon_D^*}{\gamma} + \frac{1}{\text{Sc}} \right) \sqrt{\gamma^2} \text{C*} \tag{4}
$$

when  $z = 0$ ,  $C^* = 1$  where  $C^* = \frac{C}{C}$  and  $C_0 =$  inlet concentration **0**

For all tracers utilized in wire wrap studies independent of the particular tracer  $\frac{E_{\textrm{D}}}{\gamma} \gg \frac{1}{\textrm{Sc}}$ . For reasons discussed before one expects  $\epsilon_{\text{D}}^* = \epsilon_{\text{H}}^*$ .

#### **3. RESULTS** OF **DATA** ANALYSIS **AND** ENERGY CALIBRATION

#### **3.1** Overview

Fig. **3.1** shows a comparison of the theoretical (hydraulic diameter including the effect of wire-wrap presence) **flow** split and experimentally observed flow split for the wall channels of wire-wrap bundles ranging in size from **7** to **217** pins. The theoretical flow split was obtained **by** (a) completely neglecting geometrical tolerance, and **(b)** allowing for geometrical tolerance. In reality, geometrical tolerance effect exists and must be taken into account. Following the method given **by** Hanson in Ref. 4 (appendix) if T is the diametrical tolerance across the flats of a bundle, a fraction F of this would be accomodated within the bundle and a fraction **(1 -** F) in the wall region. Then the pitch, P, of the wire wrapped bundle must be increased **by** AP due to the "looseness" caused **by** geometrical tolerance, where

$$
\Delta P = TF / (\sqrt{3} . N_{\text{PTMCS}})
$$

The gap between the wire and the wall is given **by**

$$
\Delta R = \frac{T}{2} (1 - F)
$$

The two extreme cases for tolerance distribution are Case I:  $F = 0$   $\Delta P = 0$   $\Delta R = T/2$ Case II:  $F = 1$   $\Delta P = T/(\sqrt{3} \cdot N_{\text{prNGS}})$   $\Delta R = 0$ 

The effect of these two extreme cases on flow split in the wall channel are shown in Fig. **3.1** for the MIT 61-pin bundle which has a diametrical tolerance of **0.0195** inches in a duct with a distance across the hexagonal flats of **2.560** in.

In most bundles it is expected that Case II will exist where all the tolerance is accomodated within the bundle. Observation of the (Ref. 4) M.I.T. bundle agrees with this suggestion as the pins were observed to contact the duct faces when the bundle was assembled. This was also observed in the AI **217** pin bundle (Ref. **30).** Wherever there is a doubt as to which of the two limiting cases exists within a bundle, we have shown the two limits of flow splits for wall channel corresponding to the two cases.

It is interesting to note that for the 7-pin bundle it was possible to predict both the wall channel and inner channel flow splits (see Section **7)** accurately. As the bundle size increased there appears to be a divergence between the theoretical predictions and experimentally observed data on a similar geometry. Allowing for the experimental predictions to have an associated error bar, it still appears that the theoretical flow split predicts higher axial velocities in the wall channels (and lower axial velocities in central and corner channels) than experimentally observed. If the divergence in the theoretical and experimental curves is in fact real then a correction factor on hydraulic diameter alone would not bring the two curves together. **A** bundle size effect exists in that case. If a bundle size effect exists it could be only due to a change in the basic flow field in going from a 7-pin to a 217-pin bundle.

Until this is fully resolved one needs to take into account the flow split effect while analyzing other data and in predicting temperatures for the full size bundle at power conditions. The ENERGY analysis of data (Section **7)** takes into account a flow split effect in calibrating the two constants  $\overline{\epsilon_{\text{H}}^*}$  and  $\overline{c_1}$ .

**A** thorough analysis of each set of data described in Section **7** was made using the ENERGY code. It was necessary to completely understand the experimental techniques used to obtain the specific set of data including tolerance on geometry, accuracy of measurements and the fraction of total data, in any one experiment, that is useful for calibration purposes. In addition the ENERGY code has certain inherent assumptions built into it. Our ability to analyze a certain set of experimental data was superimposed upon the expected accuracy of the data to determine a realistic error bar on  $\overline{\epsilon^*_{\text{H}}}$  and  $\texttt{C}_{\textbf{l}}$ . For example, for the FFTF geometry the <code>HEDL</code> tests yield an **s\*** from **0.015** to **0.03,** whereas the O.R.N.L. tests H give an  $\overline{\epsilon_{\rm H}^{\star}}$  from 0.028 to 0.048. Considering the fact that the **HEDL** data could only be used after the first twelve inches from the inlet;that mass balances changed continuously from inlet to exit; that there appeared to be an effect of injection concentration; that two point measurements in the same channel gave results up to **50%** different; that the boundary condition of salt injection is not a prototype boundary condition; one would tend to have more confidence in the O.R.N.L. **19** pin

heated data. However, the O.R.N.L. experiments were on a much smaller bundle and rod bowing and bundle helixing at large power input rates were observed. Since ENERGY could predict many sets of data for various power skews with almost a single value of  $\overline{\epsilon_{\text{H}}^*}$  it was decided to put more confidence on O.R.N.L. 19 pin experiments. Therefore the range of  $\overline{\epsilon_H^*}$  recommended (for **p/d ,.** 1.24, h/d **\** 48 **- 52, d \, 0.23** in.) is from **0.025** to 0.048 with a weighted mean around 0.04. As will be seen later (Section 4) such a large range in  $\overline{\epsilon_{\text{H}}^{*}}$  only slightly affects the exit temperature distribution and the maximum exit temperature of a **217** pin FFTF bundle at full power.

Tables **3.1** and **3.2** show the fuel-assembly data analyzed. Table **3.1** lists the data based on bundle size and experimental techniques and Table **3.2** based on the significant geometrical characteristics. Only data relevant to fuel assembly calculations was considered for analysis and only the data for which all required details were available was analyzed. Although the standard fuel assembly geometry is  $d \sim 1/4"$ ,  $h/d \sim 50$ ,  $p/d \sim 1.25$ , the data analyzed covers a reasonably wide range about the standard fuel assembly design in order to permit thermal-hydraulic optimization studies. No data on poison and blanket assemblies was analyzed as this is not the present goal.

Fig. 3.2 shows that the variation of  $\overline{\epsilon_{H}^{*}}$  vs  $1/(h/d)$ is linear in the range of interest. As the wire wrap nitch is increased (d/h + 0 or h/d +  $\infty$ ) the  $\overline{\epsilon_{\rm H}^*}$  vs d/h plot is expected to curve and tend to an asymptotic value for the case of no

wire-wrap. Only the natural turbulence effect would be contributing to  $\overline{\epsilon_{\text{H}}^*}$  in that case. Fig. 3.3 is a cross-plot of Fig. 3.2. It shows that the  $\overline{\epsilon_{H}^{*}}$  vs  $p/d$  curve has a maxima in between **p/d** of 1.2 and **1.3.** Fig. 3.4 shows the variation of C<sub>1</sub> (=  $\overline{v}_0/\overline{v}$ ) vs d/h. There seems to be better agreement in data at  $d/h \sim 0.02$  (h/d  $\sim 50$ ) than at larger values of  $d/h$ . The only available data for large values of **d/h (d/h >** 0.02) are the SKOK (Ref. **8)** and M.I.T **(3)** data.

For practical use the recommended value of  $\overline{\epsilon_{H}^{*}}$  is the value read from Figs. 3.2 or 3.3 with approximately a  $+30$ % (of  $\overline{\epsilon_{\text{H}}^{*}}$ ) error bar. At d/h  $\sim$  0.02 p/d  $\sim$  1.24, the range shown in Fig. **3.2** is **0.025** to 0.048. Wherever the range is not shown the above mentioned range in  $\overline{\epsilon_{H}^{*}}$  may be used. For  $C_1$  the range shown in Fig. 3.4 must be used.

The analysis of data of Ref. **8** is described in Section **7** and Table III. Since only data over one axial wire wrap pitch is given in Ref.8, our confidence level in analyzing it is not good. The values of **C1** obtained from the data of Ref. **8** required many assumptions. It appeared that **C1** so obtained was underestimated. Thus the SKOK data is shown as a lower limit on C<sub>1</sub>. The M.I.T. results on the other hand appear to overpredict C<sub>1</sub>. This is because only one value of transverse velocity in the gap between rod and wall was measured. If the flow field is turbulent, the average gap velocity could be as much as 20% lower than the point velocity measured. Superimposed upon this velocity profile effect is an independent error bar in the range 12 to 14%. The value of  $C_1$  lies between the limits shown in Fig. 3.4.

The value of  $C_1$  for the FFTF geometry lies somewhere between 0.07 from the HEDL<sup>23</sup> data to 0.12 from O.R.N.L.<sup>9</sup> and M.I.T.<sup>3</sup> data. Discussions of bundle size effect are delayed until later. However the effect of this range of **C1** on the duct temperature gradient (Section 4) does not appear to be very significant.

The calibration of ENERGY described above should hold for the **p/d** and **h/d** ratios considered. In addition all of the data (except SKOK data for which ENERGY calibration is not expected to be good) used was for rod diameters close to **0.25** in. Rod diameters in a range close to this should present no significant effect on **e\*.** This limitation is imposed as a precautionary measure (see Ref. **26;** calibration of THI-3D also imposes this limitation) since data with other rod diameters have not been analyzed.

#### **3.2** Comparison of Heated Pin, Hot Water Injection and Salt Injection Experiments

Our analysis of the various types of data showed that the heated pin experiments are probably the most reliable, provided care is taken in designing the temperature sensors and the heated rods. The discussion to follow compares heated pin experiments with non-heated pin experiments (salt and hot water injection included). The Laser Doppler Velocimeter experiments are not discussed since they can at present only be used for wall channels where they can supplement the results of any of the above experiments.

The major problem with the two non-heated experiments (apart from a different boundary condition than the prototype) is that an injector must be introduced into the main flow stream. Local perturbations of the bulk flow can affect the measurements many inches downstream of the injection point. Three factors that are extremely important in obtaining reliable reproducible results with any injection device are the injector design, injector position and injection flow rate  $12, 19$ . Considerable effort has been devoted by Lorenz<sup>19</sup> and Pederson<sup>12</sup> in testing various injector designs. Their results show a centrally located injector with injection velocities close to the local mainstream velocity should give reliable results for wall channels. An extension of a similar detailed study  $24$  for centrally located injector is required for central channels. The HEDL<sup>23</sup> inlet perturbation effects were found to last from **6** in to **9** in. The flow drift of two channels (see Section **7)** has been interpreted **by** investigators as an internal swirl flow. ENERGY analysis shows that this drift is an inlet perturbation effect. **A** similar inlet perturbation effect was found by examining, level by level, the GE-127 pin<sup>22</sup> hot water injection data. The inlet perturbation in this case is an extremely small internal swirl flow which dies off after about **12-18** in.

Every size of bundle tested needs to have a detailed injection study made similar to that in Refs. 12 and **19,** especially for wall channels. The state of our understanding of the hydraulics of the wire wrapped bundle is such at present

that an injection scheme tested for a small bundle cannot be used for a large bundle with a great deal of confidence. Thus until a complete knowledge of the local flow fields within subchannels of a wire wrapped bundle is available each bundle would require related injector development work.

The tracer detection **by** wall thermocouples (for hot water injection) and wall conductivity probes is a localized point measurement not suitable for lumped subchannel analysis computer programs. The isokinetic sampling technique<sup>6</sup>, however, along with conductivity probes (or thermocouples) located near or at the center of the subchannel<sup>9,13</sup> can yield more useful results. Although wall thermocouples can be designed $^{22}$  carefully, a steep temperature gradient between wall and the center of the subchannels could aive misleading results. The M.I.T. experiments<sup>27</sup> use a variable injection position and fixed detection probes at the outlet of the bundle. These probes are located at the center of the subchannels. The tracer detection in non-heated experiments can, therefore, be carefully designed to give reliable results. The mass balance (in salt-injection experiments) and energy balance (in hot water injection experiments) should be as good as the energy balance in heated pin experiments<sup>9</sup>. To date poor mass balances<sup>23</sup> and energy balance $^{12}$  for central injection have been obtained. Heated pin experiments (Refs. **9, 15)** have yielded energy balance within **3-5%** as compared to **13-35%23** for non-heated pin experiments.

The isokinetic sampling technique does not appear to suffer from the same major drawbacks as the others but a possible source of error in this technique is discussed below. Using the isokinetic sampling technique the differential static pressure between a subchannel and another reference subchannel at the exit plane to the test section is measured under the undisturbed conditions. Then an extraction device is placed at the measuring plane and flow withdrawn until the previous  $\Delta P_{ref}$ , between the subchannel and the reference subchannel is obtained. It is then assumed that the original flow split is re-established and the "proper flow" flow split is being sampled<sup>6</sup>. The problem with the technique is that reestablishing the previous  $\Delta P_{ref}$  between the subchannel and reference subchannel is necessary but not sufficient to ensure that the original flow split is established. This is because the pressure difference between the subchannel in question and subchannels other than the reference may not be the same as before even though  $\Delta P_{ref}$ , has been re-established. Fig. 3.5 shows a comparison of isokinetic sampling data on nearly similar geometries in a 7-pin $^6$  and in a 91-pin $^{19}$  experiment. Although wire orientations are different for the two cases the difference in results cannot be explained **by** wire-orientation effect alone since wire orientation **C** lies in between **A** and B but the corresponding curve for **C/C** does not.

It could possibly be a bundle size effect. If the latter is ruled out (at present the latter cannot be ruled out) then the reliability of the technique can be questioned. More

experiments are required to test fully the reliability of the isokinetic sampling method.

The most useful portion of the data generated **by** salt and hot water injection experiments is produced in the first twelve to fifteen inches axially. Codes like  $C\phi TEC$ <sup>11</sup> and THI-3D<sup>10</sup> use the first few inches for sweep flow calibration. As discussed in subsection X of Section **7,** as one goes downstream very small errors in tracer concentration yield a large range for  $\overline{\epsilon_{\rm H}^*}$  (the calculated salt concentration vs  $\overline{\epsilon_{\rm H}^{\star}}$  curves become flatter with distance downstream. A small error bar on data superimposed upon this could give a very wide range of  $\overline{\epsilon_{\text{H}}^*}$ .). Moreover for ENERGY purposes the injection channel data was found to give the least errors in calibration (see Fig. **8** of Appendix) for the reasons mentioned in Section **7.1.** If inlet perturbation effects continue for a few inches downstream of the injection plane, the most useful portion of the data is lost. It is more difficult to develop data free of inlet perturbations for central injection than for wall injection due to lack of maneuverability (bundle must be disassembled every time) and accessibility. In heated pin experiments any rod in the bundle can be easily heated **by** external control. Whereas concentrations in the injection and neighboring channels continuously decrease in the tracer injection experiment, the temperatures of the channel near the heated rod or rods in a heated pin experiment (line source) continuously increase. Heated pin experiments can be designed

more easily to give accurate reliable axial temperature variations **by** changing the length of the pin heated and using the same exit (see Section **5A)** thermocouples. The data from Refs. **9, 15** and **16** has generally shown a greater degree of accuracy and reliability than for tracer injection experiments.

One of the most important features of tracer injection experiments (that can again be achieved **by** heated pin experiments in which heat is generated only over a small axial length) was found in Ref. 12. Tracer injection into an inner (No. 12) channel adjacent to the wall channel (see Fig. **33** of Ref. 12) shows that most of the fluid is transferred to an adjacent inner channel **11 by** diversion cross-flow2 (due to the presence of wire in channel 12 its hydraulic diameter decreases) and very little to adjacent channel **3.** One would have expected at least an equal amount to be transferred to both channels due to diversion cross-flow plus an additional amount to channel **3 by** sweep flow as the wire crosses the gap between rods 2 and **3.** It is not correct to assume that interaction between wall and inner region is small based on this evidence. There is some **15** evidence that (Section **7)** rixing is uniform in the bundle However, a knowledge of these local hydraulic interactions between subchannels can be very useful in trying to assess the relative importance of sweep and diversion cross-flow in codes that linearly add these two effects. It is believed by the present authors that no code, as yet, can predict this type of a local flow redistribution with any degree of confidence. This is one reason that simpler codes like ENERGY prefer to lump

these modes of energy transfer. However as more sophistication is built into the hydraulic models of the more advanced codes like THI-3D and COBRA, this type of data will be most useful.

Only two sets of heated pin experiments are available for the fuel-assembly geometry. These are the ORNL<sup>9</sup> 19 pin and the Germanl6 **61** pin experiments. Although several sets of useful data are available from these experiments a number of important issues still remain unclear. The German $^{16}$  2 heated pin experiments show (Fig. **1.8)** that the effect of wall swirl flow penetrates at least  $3-4$  channels radially inwards, almost to the center. This effect is not due to conduction because conduction effects have been found to be an order of magnitude smaller than the wire wrap mixing effects for Re **> 10000.** If internal swirl exists (which we do not believe, based on our analysis of the **217** pin bundle. However an additional set of data could put the matter to rest) for the **61** pin bundle its effect cannot be separated out due to penetration of wall effects. However, if a 217-pin experiment were performed using water with single or two heated pins strategically located it would be possible to confirm our belief, based on analysis of **HEDL** experiments, that no internal swirl exists. **By** locating heated pins near the wall the swirl flow  $(C_1)$  can be determined and further confirmed **by** laser measurements. In addition the existence of bundle size effects can be checked. The need for such an experiment (for a large range of **h/d, p/d, d)** exists and is described in Section **5.**

#### **3.3** Reynolds Number Effect

Both  $\overline{\epsilon_{\text{H}}^*}$  and  $\overline{C_1}$  were found to be independent of Reynolds number for Re **> 10000.** This result is not unexpected and needs no further discussion.

#### 3.4 Bundle Size Effect

**A** question that must be resolved is whether the swirl flow and mixing coefficient (or sweep flow) remain constant as the size of the bundle is increased (more pins) for the same geometrical characteristics **(p/d, h/d).** There is every reason to expect a smaller bundle  $(\sim 61-91 \text{ pins})$  to be able to model a larger **(217** pin) bundle well if **h/d** and **p/d** remain the same. If a bundle size effect actually exists it should be more obvious in going from a **19** pin to a **91** pin bundle than in going from a **91** to a **217** pin bundle, based on ratio of interior to wall channels. However, our analysis has shown that **C1** remains almost constant in going from a **19** to a 91 pin bundle. Yet the HEDL<sup>23</sup> analysis shows that  $C_1 \sim .07$ , almost half of the swirl velocity in the small bundles (Fig. **3.6). The**  $\overline{\epsilon_{\text{H}}^*}$  **obtained from the <code>HEDL<sup>23</code> analysis is also consi-**</code></sup> derably smaller than that obtained from the ORNL $^9$  heated pin experiments. At this stage, with the available data, one cannot determine with any degree of certainty whether this is a bundle size effect or if it is an instrumentation and injection effect.

The flow split (Fig. **3.1)** shows a bundle size effect. Our analysis of the available data shows that internal swirl may be prevalent for smaller bundles (although its magnitude is

so small so as not to effect the results) but as the bundle size increases the internal swirl decreases. The ORNL<sup>9</sup> 19-pin experiments show an asymmetry in radial temperature distribution for the centrally heated pin. This could be due to rod bowing or internal swirl. The Japanese<sup>13</sup> experiments (Fig. 14) of appendix) show an asymmetry in tracer concentration at the exit of the bundle. This could be due to internal swirl. However, analysis of the HEDL<sup>23</sup> experiments shows that no internal swirl exists.

In addition Fig. **3.5** shows that the difference in concentration decay rates for wall injection into two bundles of different sizes **(7** and **91** pins) but similar geometry is quite large. Is this a bundle size effect?

Does the basic hydraulics change within a bundle in going to a large bundle? If so, data obtained on smaller bundles should not be used for large bundle predictions. This open question can only be resolved with more experimentation.

Given below is more evidence of the difficulty one faces in trying to determine if the observed differences in local flow field is due to a bundle size effect or due to different experimental techniques. In Ref. 12 Pederson has experimentally observed that subchannels near the downstream corner of a face (in the swirl flow direction) have a higher swirl flow. Based on this observation an explanation for the decrease in swirl flow with increasing bundle size is available. Fig. 3.7 shows the velocity ratio  $\overline{v}_q/\overline{v}$  (=C<sub>1</sub>) for these adjacent rod gaps near the downstream corner of a face, as a function of axial distance. These were measured **by** a Laser Doppler Velocimeter and are expected to be quite accurate **(+ 15%)** (Ref. 4). No significant change in swirl velocity is observed for any position of the wire wrap, unlike that observed **by** Pederson12 for a **91** pin bundle. It is difficult to imagine a significant bundle size effect of this type can exist in going from **61** to **91** pins for similar geometries.

#### 4. Application of ENERGY calculations to a 217-pin FFTF Bundle

The callibrations of the ENERGY code is described in section **7.** The values of the two empirical constants  $\overline{\epsilon_H^*}$  and  $C_1$  were determined from several sets of data on geometry similar to the FFTF dimensions. The value of  $\frac{1}{\epsilon_H^*}$  obtained was in a range  $0.025$  (HEDL) to  $0.048$  (O.R.N.L.) and  $C_1$  was in a range from **0.07 (HEDL)** to 0.12 (O.R.N.L., M.I.T.). Moreover there is a considerable uncertainty in flow split which must be taken into account. In order to see the effect of this wide range in the coefficients, on our ability to predict the coolant temperature distributions in a prototype FFIF 217-pin bundle a parametric study was run using the following base parameters.

**1).** The mean temperature rise across the bundle of **300\*F.**

2). Axial power skew (peak/avg.) of **1.23** and

**3).** Radial power (linear) skew (max/min) of **1.50.**

4). Inlet temperature of the coolant of **6000F.**

The maximum duct wall temperature difference at the exit of the core was investigated for various size bundles. The base case conditions were maintained for bundles of all sizes. Fig. 4.1 shows that whereas mixing (finite  $\overline{e^*}_{H}$ ) reduces (ATmax) duct for small bundles it increases it for bundles containing more than 91 pins. Thus an increased value of  $\overline{\epsilon_{\text{ff}}^*}$  is conducive to reducing the maximum ( $\triangle$ Tmax.)<sub>ax</sub>, temperature within the bundle but increases the duct wall temperature difference. The swirl flow (finite  $C_1$ ) on the other hand has been found to reduce the (ATmax) duct without any appreciable effect on (ATmax) axial since (ATmax)  $_{\text{axial}}$  occurs inside region I which is unaffected by  $C_1$ .

Fig. 4.2 shows the maximum dimensionless axial temperature rise in the

 $\frac{d}{dx}$  ] bundle and the maximum dimensionless duct wall remperature  $(ATmax)$ <sub>d</sub> difference  $\left[\frac{1}{\sqrt{2\pi}}\right]$  at the core exit as a function of  $\epsilon_H^*$  and  $C_1$ . Fig. 4.2 was plotted for the theoretical flow split case (see Fig. **1).** By superimposing the  $\overline{\epsilon_{H}}^*$  range from 0.025 to 0.048 on figure 4.2 one obtains the following range of predictions:

$$
1.286 \le \left(\frac{\Delta \text{Imax}}{\Delta \text{T}}\right) \text{axial} \le 1.307
$$
  

$$
0.273 < \left(\frac{\Delta \text{Imax}}{\Delta \text{T}}\right) \text{duct} \le 0.301
$$

**<sup>A</sup>**range similar to equation 4.1 above can be obtained for the case where uniform velocity is maintained within the bundle. The following range of predictions results:

1.250 
$$
\leq \left(\frac{\Delta \text{Trans}}{\overline{\Delta \text{T}}}\right)_{\text{axial}} \leq 1.27
$$
  
0.291  $\leq \left(\frac{\Delta \text{Trans}}{\overline{\Delta \text{T}}}\right)_{\text{duct}} \leq 0.318$  4.2

Based on the limits specified **by** equations 4.1 and 4.2 the following limit on predictions of the maximum temperature within the bundle, Tmax, and  $(\mathcal{A}_{\text{max}}^T)$  duct is formed.

$$
977^{\circ}F \leq \text{Imax} \leq 993^{\circ}F
$$
  
82<sup>°</sup>  $\leq$  ( $\triangle$ Imax)  
core exit

The limits on Tmax are clearly shown in Fig. 4.5 where temperatures along a cross-section A-B of the bundle exit (exit of core) are shown. As  $\overline{\epsilon_{\text{H}}}^{\star}$  is decreased, the maximum exit temperature increases and occurs at a position closer to the duct wall.

A large range of  $\overline{\epsilon_{H}}^*$  and  $C_1$ , do not significantly effect Tmax and  $(\Delta T_{\text{max}})$  duct. Thax, can be predicted to within  $\pm$  8°F and ( $\Delta$ Thax)  $_{\text{duct}}$  a the core exit to within  $\pm$  6.5° F. It is interesting to note that most of the predictions **by** other codes lie very close to this range (Table **4.1** and Fig. **4.5) .**

It is necessary to indicate that the range of  $\pm$  8° F on a  $\Delta T_{\text{axial}}$  of **<sup>2850</sup>**F (or **885<sup>0</sup> <sup>F</sup>**for Tmax at exit) may be deceptively small especially since hot channel factors have not been inluded in the calculations. With the inclusion of H. **C.** F' s the temperature range within which we can predict Tmax with great degree of confidence is expected to increase.

Fig. 4.6 shows the variation of  $(\Delta \text{max})_{\text{duct}}$  with axial distance for two values of **C1 .** The axial distance is measured from the inlet of the core (active-fuel) and extends to the exit of the fuel-assembly. Fig. **4.7** shows a cross plot of Fig. 4.6 for the nominal  $\overrightarrow{e_H}^* = 0.04$ . It is seen that the effect of lowering C<sub>1</sub> from 0.12 to 0.07 is to raise the spatial average value of (ATmax)<sub>duct</sub> from 36 in. to 84 in. by approximately 10°F. Duct structural designers must specify if this difference in  $(\triangle \text{Tmax})$  duct is significant.

The peak temperature in the duct wall at the 84 in elevation occurs at the midpoint of face C (see Fig. 4.4) for  $C_1 = 0.12$ . However if  $C_1 = 0.070$ the peak temperature occurs on face **A** very close to the corner between faces **A** and **C.** The rotation of the peak temperature in the duct wall appears to be relatively insensitive to **C1 .** This should be useful for core design restraint purposes.

#### **5.1** Proposed Experiments

Based on the data analysis (Section **7),** discussions (Section **3)** and application of the code ENERGY to a 217-pin prototype bundle at full power (Section 4) we have come to the conclusion that further experimentation is required if reduction of the range of  $+$  8°F on  $T_{max}$  and  $+$  6.5°F on  $(\Delta T_{MAX})_{D U C T}$  is desired. The reduction in the space-averaged  $(\Delta T_{\text{max}})$  duct was about 10°F when  $C_1$  was increased from 0.07 to 0.12. Since each degree F reduction in  $T_{max}$  could mean an increase in max. burnup **by** up to **500** Mwd/T **(3)** it is imperative that a better understanding of the flow field and bundle size effects be obtained from more carefully designed experiments. Two different experiments will be briefly described. The first experiment is similar to the heated pin experiments described in Ref. **15.** The second experiment has not been used before for LMFBR mixing studies and is strongly recommended. It may even be possible to combine the two types of experiment in an optimum fashion into a single setup. Before describing the experiment design let us again briefly examine the need for these experiments.

> **(1)** Each \*F reduction in Tmax could mean up to **<sup>500</sup>** Mwd/T increase in burnup. Decrease in  $(\Delta T_{max})_{duct}$  also means an increase in burnup.

> (2) Tables **3.1** and **3.2** show that certain data gaps exist before a thermal-hydraulic optimization study on LMFBR fuel assembly design can be made. Very little

reliable data exists for swirl flow and  $\overline{\epsilon_{\text{H}}^*}$  at low values of **h/d (h/d <** 48). The only complete set of data available below  $h/d < 24$  is SKOK<sup>8</sup> hot water injection data on 7-pin bundles. More data is required on larger bundles for low **h/d** and **p/d** and different rod diameters.

**(3)** Although several sets of experiments have been performed on FFTF bundle geometry, yet there is a great deal of speculation with regards to the flow field in bundles of various sizes. For example, is the bundle size effect observed real or is it due to faulty instrumentation and errors in measurement? Is the flow split a function of bundle size? **A** knowledge of the correct flow split could significantly reduce errors in predicting temperature distributions. If internal swirl flow exists for smaller bundles and reduces to zero as the bundle size increased to a **217** pin bundle, then the internal swirl flow must affect the flow field in these smaller bundles. Therefore experimental data observed on bundles smaller than the full size bundle could lead to erroneous results. Ref. 12 has found that the mixing coefficient between wall and the interior channel is low. Conflicting evidence exists on this. It must be resolved.

None of the experiments on heated pin measure fluid temperatures in the axial direction. Point measurements of temperatures **by** the wire and wall thermocouples cannot be easily used **by** existing subchannel analysis codes. The experiment proposed will permit measurement of axial fluid temperature distribution with as much accuracy as the exit fluid temperature distribution **by** varying the heated length of the pin but keeping the same exit where the exit rake thermocouples measure the fluid temperature.

#### **5.2** Experiment I

**<sup>A</sup>***217* pin experiment is proposed below with single or two heated pins strategically located in the manner suggested in Ref. **16. A** wide range of **d, h/d** and **p/d** should be covered for two bundle sizes **(91** and **217** pin). **A 217** pin bundle is shown in Fig. **5.1.** Fig. **5.1** also shows the location of the thermocouples in the subchannels. These thermocouples are placed in an exit rake similar in design to the O.R.N.L. exit rake. Water is used as the coolant. The use of water as coolant is justified for fuel assemblies where buoyancy effects are unimportant at high Reynolds number and also (since  $\overline{\epsilon_{\rm H}} >> \alpha$ ) thermal conduction is unimportant both for sodium and water. Initially only rod **1** is heated. Fig. **5.2** shows the temperature rise as a function of the radial distance for three different Reynolds numbers. The rod power for the experiment in Fig. **5.2** is 10Kw/ft. Since  $\overline{\epsilon_{\text{H}}^*}$  is not expected to be a function of Reynolds number, the same value of  $\overline{\epsilon_{\text{H}}^*}$  ( $\overline{\epsilon_{\text{H}}^*}$  = 0.04) was used to obtain curves I, II

and III. Fig. **5.3** shows the temperature distribution vs radial distance when the rod power is changed to **5** and **15** kw/ft but Reynolds number is maintained at **10000.** One would ideally like to use a rod with the maximum power rating. But rod bowing at high linear power rating (i.e. **8** kw/ft) has been observed <sup>9</sup> in a **19** pin experiment. Here the power to flow ratio is much smaller than in Ref. **9** and it is expected that bowing and related problems should diminish considerably in severity. In order to cover a wide Reynolds number range **(5** x **103** - **15** x **10 3** and yet produce accurate data with the lowest power rating of the rod it is recommended that the **10** kw/ft rod be used for the experiments. As the h/d is reduced the mixing rate  $\overline{\epsilon_{\text{H}}^{*}}$ increases and the maximum temperatures in Fig. **5.2** would reduce. The thermocouples can be designed to measure temperatures to within +  $1/2^{\mathbb{O}}$ F. Since only one thermocouple (located at the subchannel centroid) is used in each subchannel in these experiments it is expected that the temperature measured **by** it may be slightly different from the subchannel average temperature. However, the temperature gradients are small in the transverse direction and the difference in the thermocouple reading and the subchannel average temperature is expected to be small. In order to take such errors into account for determining a realistic estimate of the reduction in the range of  $\overline{\epsilon_H^*}$  and  $C_1$ from the proposed experiment a large error bar of + 1°F is applied to the thermocouple precision. Fig. 5.4 shows the calculated variation of the temperature rise recorded **by** thermocouple L vs  $\overline{\epsilon_{\text{H}}^*}$ . On superimposing a  $\pm$  1°F error bar on these curves one finds that  $\overline{\epsilon_{\text{H}}^*}$  can be determined accurately.
As the power to flow ratio decreases (in going from curve I to III in Fig. 5.4) the precision with which  $\overline{\epsilon_{\text{H}}^{\star}}$  can be determined decreases. For this reason curve III in Fig. 5.4 shows a much larger  $\overline{\epsilon_{\rm H}^*}$  range than curves I and II. Therefore for the **10** kw/ft experiment only curves I and II (at Reynolds number of 5000 and 10000) should be used to determine  $\overrightarrow{\varepsilon_{H}^*}$ . A third set of data, if desired, could be obtained at a Reynolds number of 7500. The range within which one can determine  $\epsilon_{\text{H}}^*$ (assuming a nominal value of  $\overline{\epsilon_{\text{H}}^*}$  of 0.041) is 0.038 to 0.044.

Fig. **5.5** shows the radial temperature distribution for two rods heated one at the center and one at the wall (Fig. **5.1).** Both rods have the same power. It is seen that the swirl flow affects the temperature gradients significantly only near the wall. The wall thermocouples  $F_1$  and  $F_2$  show a large effect of variation of **C1 .** This is also seen in Figs **5.6** and **5.7** where the circumferential temperature profile is plotted.

As the swirl velocity ratio  $C_1$  is varied, calculations show that the temperature profile in the circumferential direction changes significantly. Fig. **5.8** shows the calculated sensitivity of the temperature of thermocouple  $F_2$  to variations in **C1 .** It is obvious that **C1** can be determined to a high degree of accuracy. An independent set of measurements using the Laser Doppler Velocimeter (LDV) can be used to verify the value of C<sub>1</sub> so obtained. The experience at M.I.T. with the LDV has shown that **C1** can be obtained within an accuracy of 12 to 15%. ENERGY analysis of the ORNL 19 pin experiment<sup>9</sup> showed that  $C_1$  lies in the range  $0.12 - 0.14^1$ . Much later it was verified<sup>4</sup> **by** Laser Measurements that for a **61** pin bundle of a similar geometry  $C_1 = 0.13 + 0.015$ . It is thus assumed that  $C_1$  can be determined (if the nominal value is 0.12) within a range **0.11** to **0.13.** The LDV should also be able to give valuable information on the flow split. Assuming the flow split is known and that  $\overrightarrow{\epsilon_{\text{H}}}$  is determined to be in the range 0.038 to 0.044 and **C1** is within the range **0.11** to **0.13,** one can show, using Fig. 4.3 (or Fig. 4.2), that the maximum core exit temperature,  $T_{max}$ , can be predicted to within  $\pm$  2°F and the  $(\Delta T_{max})_{DUCT}$  can be predicted at the core exit to within **+ 2.5\*F.** Even if these limits were arbitrarily doubled, the precision with which the temperature distribution can be determined from the proposed experiments would be far better than the present day accuracy and confidence of these predictions.

The axial temperature distribution can also be determined from the proposed experiment **by** using different heated rod lengths (linear power rating, Reynolds number etc. should be left unchanged) and measuring the temperatures at the exit of the bundle. Thus **by** varying the active heated length (measured from the exit of the bundle) the axial temperature distribution can be obtained.

## **5.3** Experiment II

This experiment is an integral part of experiment I and provides information that can be input into the first experiment.

The motivations for this experiment are as follows. There is a considerable concern and some experimental evidence (Ref. 12) that the hydraulic interaction between the wall and the first row of inner channels is smaller than the hydraulic interaction between inner channels. In terms of ENERGY nomenclature, there appears to be evidence that  $\overline{\epsilon_{\text{H}}^*}$  is smaller at the boundary of the central and wall regions than its value in the inner region. This must be resolved.

Another motivation which is, perhaps, even more important than the first is as follows. While calibrating computer programs like **COTEC11and** ENERGY1 a certain velocity profile is assumed in the bundle. Codes like THI-3D $^{10}$  calculate this velocity profile. While some small amount of data is available on flow splits there is not a single set of reliable data that can give the cross-sectional distribution of subchannel average velocities in the bundle. If the  $\c{codes}^{1,11}$  assume a 3-dimensional velocity profile or if THI-3D $^{10}$  calculates a velocity profile different from the 'correct'velocity profile there will be errors made in calibration of mixing coefficients. Moreover the velocity profile may vary axially due to changes in coolant properties with temperature. Thus substituting a known velocity field into the energy equation and then determining a  $\overline{\epsilon_{\rm H}^{\star}}$  (or a coefficient for sweep flow as was done for THI-3D when using ORNL data(Ref.26)) **by** matching temperature fields is not strictly correct.

The experiment described below does not use the fluid within the bundle as a heat sink and so the velocity and temperature profiles remain constant axially. For such a system with no internal heat generation the energy equation becomes

$$
\frac{1}{2} \left[ r \left( \rho C_p \epsilon_H(r) + k \right) \frac{3T}{2} \right] = 0 \tag{5.1}
$$

At high Re for water (also for sodium) as coolant in the presence of wire wrap  $\rho C_p \epsilon_H$  >>  $k$ , then

$$
\frac{\partial}{\partial r} [r \rho C_p \epsilon_H \frac{\partial r}{\partial r}] = 0 \qquad (5.2)
$$

If the total heat transferred in a length L is  $Q(Btu/hr)$  then using this as a boundary condition

$$
\varepsilon_{\rm H}(\mathbf{r}) = -\frac{\Omega}{2\pi r \mathcal{L}(\rho C_{\rm p}) (\frac{\partial \mathbf{T}}{\partial \mathbf{r}})}
$$

whence

$$
\varepsilon_{\text{H}}^{\star}(\text{r}) = - \frac{Q/(\overline{V} \text{d} \text{e})}{2 \pi r (\rho C_{\text{p}} \text{L}) (\sum_{\text{T}}^{\text{d}} \frac{\text{d} \overline{T}}{\text{r}})_{\text{r} = \text{r}}}
$$
(5.3)

$$
\overline{\mathbf{V}} = \text{bundle average velocity}
$$

This type of experiment was originally designed **by** Yagi and Kunii (Ref. **29)** for studying heat transfer near wall surface in packed beds. If **Q** is known then **by** measuring the temperature gradient  $\left(\frac{\partial \Gamma}{\partial r}\right)_{r=r}$  at the radial position 'r' one can calculate the radial distribution of  $\epsilon_H^{\star}(r)$ . Thus, if the mixing coefficient near the duct wall decreased, it should be reflected **by** a change of slope of the T vs r curve. With such a system, therefore, the velocity data need not enter the calculation of  $\overline{\epsilon_{\text{H}}^*}$  and consequently the possible error in determining  $\overline{\varepsilon_{\text{H}}^{*}}$  is reduced.

Figs. **5.9** and **5.10** show the cross-section of the experimental setup. The details, such as flow rates within the bundle and the coolant flow rate outside the bundle, can be calculated easily and are not given. The thermocouple locations are also not shown in the figure.

Since the fluid phase is not a heat sink a steady temperature profile (a single heated rod is located at the center of the bundle) distribution will be established after a developing length which could be as long as **8-9** ft. It would be possible to reduce considerably the axial distance required to achieve steady temperature profiles provided preheated fluid with a similar temperature distribution as the steady state distribution could be supplied at the inlet of the bundle. Obviously one does not know the steady temperature profile 'a priori'. So to start the experiment, preheat the inlet coolant to give a radial inlet temperature distribution (in practice several concentric zones) which is similar to a calculated steady temperature profile. Then **by** a trial and error procedure it should be fairly straightforward to preheat the inlet fluid in a manner such that steady temperature profile is established within 12 to **18** in.

It is necessary to note that a one-dimensional energy transport is assumed in the formulation of equation **(5.3).** Consideration must be given to the fact that a swirl velocity exists near the wall.

However, for the centrally heated rod it is difficult to imagine the existence of a circumferential temperature gradient. The German experiments<sup>15,16,17</sup> do not report a temperature gradient in the circumferential direction for the case when the central pin was heated. These tests were carried out very carefully, at low power rating of the centrally heated

pin. The O.R.N.L. **19** pin experiments report a circumferential temperature gradient for the centrally heated pin. It is believed that this is due to rod bowing since the pin was operated at **10** kw/ft. and bowing was observed at power ratings above **8** kw/ft. Even if a very small fraction of the total energy is transported circumferentially (which we believe will not occur after 12-24 in.) the results should not be affected to any large extent.

The sequence in which the two experiments should be conducted is as follows:

**1)** Perform the single centrally heated pin portion of Experiment I and determine if circumferential temperature gradients exist.

2) Perform the two heated pin portion of Experiment I **-** get swirl flow data. (For this case uniform  $\overline{\epsilon_{\rm H}^*}$  is assumed throughout the bundle.)

**3)** For the pin-array of Experiment I **-** get swirl flow data from Laser Doppler Velocimeter.

4) If in step **1** it is found that for the centrally heated pin circumferential gradients exist, Experiment II cannot yield  $\varepsilon_{\text{H}}^{*}(\text{r})$ . If however circumferential gradients are found to be very small or non-existent then conduct Experiment II to determine  $\overline{\epsilon_{\text{H}}^{*}}(r)$ . The bundle array of Experiment I could be modified **by** adding a circumferential cooling jacket. **5)** It is expected that  $\overline{\epsilon_H^*}$  is not a function of radius. However, if it is found to be a function of the radial position in

Ĵ

step 4, step 2 must be repeated and in calibration of **C1** the swirl velocity ratio, the correct local  $\overline{\epsilon_{H}^{*}}$  should be used.

## Titles For Illustrations

 $\ddot{\phantom{a}}$ 

## Figure No.



 $\sim$ 

 $\frac{1}{2}$ 



 $\frac{1}{4}$ 



TABLE **3.1**

 $\mathcal{L}(\mathcal{L}^{\mathcal{L}})$  and  $\mathcal{L}^{\mathcal{L}}$  are the set of the set of the set of  $\mathcal{L}^{\mathcal{L}}$ 

 $\mathcal{L}^{\mathcal{L}}(\mathcal{L}^{\mathcal{L}}(\mathcal{L}^{\mathcal{L}}(\mathcal{L}^{\mathcal{L}}(\mathcal{L}^{\mathcal{L}}(\mathcal{L}^{\mathcal{L}})))$ 

 $\mathcal{L}$ 

**LEGEND C** 1,2, **--- <sup>11</sup>NUMBER ASSIGNED** TO **EACH** EXPERIMENT FOR **USE IN** TABLE **3.2 (** TYPE OF EXPERIMENT **(a)** HOT WATER **INJECTION (b) SALT INJECTION**

> $\mathcal{L}^{\text{max}}_{\text{max}}$  and  $\mathcal{L}^{\text{max}}_{\text{max}}$ **Contract Contract**

 $\mathcal{L}^{\text{max}}_{\text{max}}$  and  $\mathcal{L}^{\text{max}}_{\text{max}}$ 

(c) **HEATED PIN (d)** LASER

 $\mathcal{L}^{\text{max}}_{\text{max}}$  and  $\mathcal{L}^{\text{max}}_{\text{max}}$ 



TABLE **3.2 DATA ANALYSED**

**\* NUMBERS** REFER TO TABLE **- I**

 $\sim$ 

 $\sim 10^7$ 

 $\sim$ 

## Table 4. **1 (19)**

Comparison of Predicted Maximum Subchannel Temperatures at the Core Exit for a 217-pin FFTF Assembly With a **1.** 2 Power Skew and **300** Core **AT.**

Max. Coolant Temperatures



 $\mathbf{I}$ 



**FIG. 3.1**



**FIG. 3.2**



**FIG. 3.3**







**FIG. 3.5**



FIG. 3.6





FIG. 3.7



FIG. 4.1



FIG. 4.2



FIG. 4.3



FIG. 4.4







FIG. 4.6



FIG. 4.7





 $\mathbf{\hat{z}}$ 

RAKE THERMOCOUPLES HEATED LENGTH = 36 IN.



FIG. 5.2







FIG. 5.4







FIG. 5.6

 $\mathbf{I}^{-1}$ 







FIG. 5.8





 $\boldsymbol{\gamma}$ 

 $\mathbf{r}$ 

 $\mathcal{P}_{\mathcal{A}^{\mathcal{A}}}$ 



 $\sim 10$ 

 $\label{eq:2.1} \frac{1}{\sqrt{2\pi}}\frac{d\mu}{d\mu} = \frac{1}{2\pi}\frac{d\mu}{d\mu}$ 

 $\mathcal{L}(\mathcal{L})$  and  $\mathcal{L}(\mathcal{L})$ 

 $\frac{1}{2}$ 

 $\mathcal{A}^{\mathcal{A}}$ 

**FIG. 5.10**

 $\bar{t}$ 

 $\mathcal{L}^{\text{max}}_{\text{max}}$ 

 $\mathcal{A}_\mathrm{c}$ 

 $\hat{\boldsymbol{\beta}}$ 

 $\bar{z}$  ,  $\bar{z}$
# Nomenclature

 $\label{eq:2} \begin{split} \mathcal{L}_{\text{max}}(\mathcal{L}_{\text{max}}) = \mathcal{L}_{\text{max}}(\mathcal{L}_{\text{max}}) \end{split}$ 

 $\bar{\beta}$ 

 $\mathcal{A}^{\mathcal{A}}$ 

 $\hat{\boldsymbol{\beta}}$ 



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# APPENDIX

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### **7A.** Data Analysis and Calibration of ENERGY

Our primary concern at present is the analysis of data relevant to fuel assemblies. Although the current typical LMFBR fuel assembly has a  $p/d \sim 1.25$  and  $h/d \sim 50$ , the data analyzed covers a large variation about these geometrical parameters. This should be useful in thermalhydraulic optimization studies.

Table I shows the various types of experiments used to obtain data. This table is ordered in terms of increasing bundle size since the bundle size "effect" is of major concern. Table II lists the same data in terms of the two most important parameters characterizing a wire wrapped fuel rod assembly. The data analysis will be presented **by** increasing size of the bundle.

### **1\*. ANL-CT-7** pin Bundle (Ref. **6)**

Lorenz and Ginsberg performed a 7-pin wire-wrap experiment with salt (NaCl) injection into upward flowing water. The subchannel flows are sampled isokinetically at a fixed measuring point located near the exit of the test section. Salt can be injected at various axial distances from the fixed measuring plane. The salt concentration of the extracted flow is measured using in-line conductivity cells. The authors claim to have obtained reliable bulk concentration data within the first spacer pitch. In addition, axial subchannel flow rate measurements were made for both wall and central channels. As pointed out **by** Lorenz and Ginsberg the isokinetic sampling

\*Numbers according to Table I

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technique permits measurement of bulk (subchannel-average) properties, thereby yielding more meaningful results for use in subchannel analysis codes than if discrete point wise measurements were made.

Fig. **1** shows the subchannel flow rates at three different Reynolds numbers. There was no obvious Reynolds number dependence on flow rates. The subchannel flow rates are expected to be periodic (with different periodicity for central and wall channels). As indicated **by** Lorenz the reason for central subchannel (#2) data falling on a straight line is that measurements of subchannel flow rates were taken every 1/6th spacer pitch at positions where the wire-wraps were in the gaps. At these locations, one would expect little variation in central subchannel flow rates. As also seen in Fig. **<sup>1</sup>** Lorenz could predict the axial average subchannel flow rates **by** theoretical flow split predictions assuming bare rods. Fig. 2 shows the variation of concentration with axial distance. Only one set of data for injection into a wall channel has been reported.

# ENERGY **-** Analysis and Calibration

Since data for only one wall channel was reported the two coefficients  $\overline{\epsilon_{\text{H}}^*}$  and  $\mathcal{C}_1$  could not be determined from the single set of data. However Novendstern's method for determining flow split in wire-wrapped bundles was used to see if the data reported in Fig. **1** can be predicted. The model of Ref. **7** has been programmed into the computer code DROP. Ref. **7** predicts

that the ratio of velocities in the central, wall and corner channels to the bundle average velocity for the Lorenz 7 pin bundle (without tolerance) is **0.9336, 1.08** and **0.8627** respectively. The ratio of the flow rates for these respective channels to the total flow rate is 0.042, **0.0977** and **0.0267.** Noting that the 7-pin bundle of Lorenz combines the wall and corner channels into their peripheral channel, one finds that the ratio of flow rates in the central and peripheral channels to the total flow rate is 0.042 and 0.1244 respectively. This is almost exactly the flow division obtained experimen-A tally. If the coefficients A and B in the expression  $f = \frac{1}{R}B$ are different than those used **by** Novendstern **,** a different flow split will be predicted. This is perhaps why Lorenz found the natural flow split assuming bare rods to predict the data well. As will be seen later only the **7** pin bundle flow split can be predicted **by** the theoretical model. As the bundle size increases the theoretical and experimental flow splits appear to diverge (Fig. **3.1).**

The attenuation of the concentration ratio, **C/C ,** for the injection channel of the 7-pin bundle is shown in Fig. 2. A comparison of the attenuation rate of  $C/C_{\alpha}$  for the peripheral channel of a **91** pin bundle, of same **p/d** and **h/d** ratios is discussed later in this section under subsection **8.** Values of  $\overline{\epsilon_{\rm H}^*}$  and C<sub>1</sub> could not be obtained from the single set of data reported.

#### 2. Battelle **- 7** pin Laser Study (Ref. **5)**

The Battelle **- 7** pin Laser study was performed on a bundle with **h/d** and **p/d** of nearly the same order of magnitude as most of the other bundles in our investigation. However the diameter of the pins was **0.866** in. Three different Reynolds numbers were used in the turbulent flow regime. No information on flow split can be derived from this set of data since the spatial velocity distribution across a subchannel was not measured. However one of the measurements made of direct use to us is the ratio of the circumferential velocity in the gap between the rod and the duct wall to the average bundle velocity. This ratio when averaged over the circumference of the bundle is the coefficient  $C_1$  in ENERGY. The value of C<sub>1</sub> reported in Ref. 5 is approximately **0.18.**

# **3.** Cadarache 7-pin Hot Water Injection Study (Ref. **8)**

The 7-pin hot water injection study was designed to investigate the effect of Reynolds number, helicoidal pitch of the spacer wires,h,and **p/d,** the ratio of the distance between pin centers to the pin diameter. Table III gives the characteristic dimensions of the bundles studied. Cold water at **68<sup>0</sup> F** was circulated upward in an hexagonal can **80** cm. long containing the wire wrapped rod bundle. Hot

water at about 140°F was injected into one of the subchannels at a variable level. In Fig. **3** the orientation of the spacer wire at the injection point was always the same. At the outlet of the pin bundle the temperatures were measured **by** thermocouples at the center of these channels.

The details of the thermocouple design are not given and hence it is difficult to estimate the range of error about the mean measured temperature. The data was not taken at a distance more than one wire wrap pitch downstream of the injection plane. It is assumed that the rods were hollow, thereby permitting very little or no heat transfer through them. Figs. **3** and 4 show the plot  $T - T_c$ of the dimensionless temperature attenuation ( $\frac{C O 10}{T - 2T}$  $hot$  $color$ for the injection channel vs axial distance. It can be seen that this temperature attenuation rate depends upon the location of the wire wrap with respect to the injection point. For example, at the entrance the wire is located at approximately the five o'clock position with respect to channel **1** and about to enter it. However the wire is located at the **6:30** position with respect to channel 4 and is not entering it at the injection plane. The experimenter found **by** hot water injection into each channel of bundle 2 that the minimum attenuation of temperature (minimum mixing rate) took place for injection into channel **1** and the average attenuation of temperature was for injection into channel 4. Similar temperature maps are available for both the donor and injection channels for injection into average and minimum subchannels for each bundle.

#### ENERGY **-** Analysis and Calibration

The ENERGY analysis of the data was basically limited to the analysis of the injector channel data. The reason for this is qiven below. ENERGY smears (axially-averaged) the effect of wire-wrap into an enhanced effective eddy diffusivity  $\overline{\epsilon_{\text{H}}^*}$ . For the injector channel every wire traverse **(3** traverses per lead length) of the subchannel causes mixing between the uniform cold temperature of the adjacent channels and the hot injection temperature and, hence, degradation of the injection channel temperature. Thus the injector channels could be simulated better within the limitations of the ENERGY model than the channels adjacent to the injector channel.

The ENERGY analysis of the data proceeds as follows. For interior injection it was found that the mixing coefficient  $\overline{\epsilon_{\text{H}}^*}$  was nearly independent of  $C_1$ . Figs. 3 and 4 show the  $\overline{\epsilon_{\text{H}}^{*}}$  values and the predicted temperature attenuation rate. As discussed before for bundle 2 the data in Fig. 3 yields the average  $(\overline{\epsilon_{H}^{*}})_{min}$  whereas the data in Fig. 4 yields the  $(\overline{\epsilon_{H}^{*}})_{\text{avg}}$ . In this manner the  $(\overline{\epsilon_{H}^{*}})_{\text{avg}}$  for bundles **1** through **5** were determined for the interior channels. It is seen that  $\overline{\epsilon_{\text{H}}^{\star}}$  is very sensitive to the temperature attenuation rate and small errors in experimental measurements can lead to large errors in  $\overline{\epsilon_{\rm H}^*}$ . Assuming a **+** 1\*F accuracy on coolant exit temperature measurement it was found that for bundle 2 for interior injection  $(\overline{\epsilon_{\text{H}}^{*}})_{\text{min}}$  = 0.033  $\pm$  0.01 and  $(\overline{\epsilon_{\text{H}}^{*}})_{\text{avg}}$  = 0.083  $\pm$  0.02.

The wall channel analysis is complicated **by** the fact that there are two unknowns  $\overline{\epsilon_{H}^{*}}$  and  $C_1$ . It was assumed that  $\overline{\epsilon_H^*}$  was uniform in the bundle. Then  $C_1$  was varied until the temperature attenuation data and the predictions matched for wall channel injection. Due to the many assumptions made in the analysis of the wall channels the values of  $C_1$  obtained would not be representative of the true swirl flow in the bundle and large error bars would be associated with the results (approx. **+ 50%).** For this reason the values of  $C_1$  so determined must be further verified **by** more experimental data. Table III shows the  $\overline{\epsilon_{H}^{*}}$  and  $C_1$  values for the various bundles analyzed. It was further confirmed **by** these experiments that  $\overline{\epsilon_{\text{H}}^*}$  is not a function of Reynolds number for highly turbulent flows. An interesting result from the data on bundles **1, 3** and **5** shows that there is a maximum in mixing rates for **p/d** in the range 1.14 **< p/d < 1.355.** However the exact **p/d** for which this maximum occurs cannot be determined from SKOK's data. No information on internal-swirl flow could be obtained from analysis of this data.

#### 4. **ORNL - 19** pin FFM-IIA Bundle (Ref. **9)**

The Oak Ridge National Lab. FFM-IIA data was taken in a **19** pin bundle with the geometry similar to the FFTF bundle. Fig. **5** shows a cross-section of the wire-wrapped bundle. Liquid sodium was used to cool the electrically heated pins. The heated length of the rods is 21 in. The coolant enters the bundle approx. **6** in. below the plane where the rods are heated in order to provide a fully developed flow field at the entrance to the heated section. Thermocouples are located in the rod walls and also in the wire wraps. An exit rake is located **3** in. above the exit of heated bundle and is instrumented with thermocouples which directly measure coolant temperatures at the center of various subchannels. The axial power profile is uniform. The hexagonal duct wall is surrounded **by** guard heaters to minimize heat losses.

Many data sets are reported for various power skews (Fig. **5)** and Reynolds numbers from about **1000** to **70000.**

### ENERGY Analysis and Calibration:

The analysis of the ORNL data was performed first for the case of all heated pins at various power skews. The ENERGY equations were solved with the following additional assumptions:

a)  $k$ ,  $\rho$ ,  $C_p$ ,  $u$  were evaluated at the average temperature over the flow cross-section and taken as uniform at each axial position; they varied and were re-evaluated at successive axial positions.

b) A uniform value of  $\overline{\epsilon_{\text{H}}^{\star}}$  was assumed throughout the bundle.

For each of the test runs the best  $\overline{\epsilon_{H}^{*}}$  and  $C_1$  was determined **by** choosing the magnitudes which gave calculated exit fluid temperature distribution that agreed with the measured value. Fig. **6, 7, 8** show the data and the calculated temperature profile. It was found that C<sub>1</sub> only affects the edge row temperatures. Therefore  $\overline{\epsilon_{\rm H}^*}$  was evaluated directly **by** using the data within the bundle. Then **C1** is varied until the best agreement with data is obtained in the outer region. The magnitude of  $\overline{\epsilon_{\rm H}^*}$  is shown in Fig. 9. It was found that both  $\overline{\epsilon_{\rm H}^*}$  and  $\overline{\epsilon_{\rm H}^*}$  are independent of Reynolds number for Re  $\geq$  15000. The optimum values of  $\overline{\epsilon_{\text{H}}^*}$  and C<sub>1</sub> were found to be  $0.04 \pm .01$  and  $0.12 \pm .02$  respectively from various sets of data where all pins are heated.

It is interesting to note that the same  $\overline{\epsilon_{\text{H}}^*}$  matches data for various power skews. This result was anticipated and confirms our physical interpretation of  $\overline{\epsilon_{\text{H}}^*}$  as a parameter dictated **by** bundle geometry.

The major problem encountered was in analyzing the single heated pin data. As discussed in Ref. **1** (see Fig. **10)** computer programs like THI-3D and **COTEC** have also not been able to predict this data satisfactorily. The single pin tests, however, raise several interesting questions, which cannot be fully resolved without additional experimental data. The large asymmetry in radial temperature profile

for a centrally heated pin can be attributed to either an internal swirl flow (i.e. due to only 2 rows of pins the wall circumferential flow is able to renetrate almost to the center of the bundle) or due to bowing of the heated pin. The single pin tests were conducted at **8-10** kw/ft. Distortion of the FFM bundle under power and flow has been observed via X-ray photography and image enhancement technique. The distortion appeared to be power **<sup>9</sup>** dependent **.** These radiograph tests were made above **8** kw/ft, at which power level the spacing between the wall and the rod appeared to vary in the axial direction. On one side of the bundle the rod to wall gap increased and on the opposite side it decreased. This change of wall to rod gap in the circumferential direction should not affect **C1** since the latter is a space average quantity. However, if inidvidual heater pins distorted in the axial direction at these power levels, then asymmetry in radial temperature distribution for a centrally heated pin should be expected.

Only for large internal swirl flow can such severe asymmetries in temperature exist. If large internal swirl exists then the inherent assumptions in ENERGY should prevent it from matching the radial temperature profiles for the 20% and **300%** power skew cases. For this reason it is believed that if internal swirl exists at all it is small in magnitude and it does not affect temperature distribution at the exit of the bundle. It is our interpretation that the severe radial temperature asymmetry for the

centrally heated pin run is primarily due to rod bowing. Other single heated pin runs were also analyzed. The runs with single heated pins at the wall were also used to determine  $C_1$ . The predictions for these runs matched with data much better than for the centrally heated pin case.

The accuracy of the coolant thermocouple was assumed to be **+ 1.5<sup>0</sup> F.** The thermocouples located in the rod walls and in the wire wrap gave a local temperature which was difficult to use to compare with ENERGY predictions of subchannel averaged temperatures.

One subchannel temperature (marked XX in Fig. **5)** was overpredicted **by** all **U.S.** codes including THI-3D for the uniform power case. No reasons for this are obvious and it needs to be resolved.

#### **5.** M.I.T. **61** pin Laser Study (Ref. 4)

The M.I.T. **61** pin bundle is being instrumented for salt-injection tests. In the meantime a Laser Doppler Velocimeter (LDV) is being used to measure the axial and swirl flow in the wall channels. Two different wire-wrap leads of **6** in. and 12 in. have been tested. LDV is probably the most accurate means of determining point velocity measurements within subchannels.at present.

Fig. lla shows the portion of the **61** pin bundle tested along with the points at which the axial and transverse velocities were measured over two wire wrap pitches.

In addition, detailed point velocity measurements were made within the subchannel which has the points **C** and B on its boundaries. The tests were performed both in the laminar and turbulent flow regimes.

Fig. **llb** shows the fully developed axial velocity profile over the wire wrap pitch **6-8** in. from the inlet. The abscissa shows the location of the wire wrap (see numbered rod in Fig. lla) with respect to the channel. The ratio of the average velocity in the wall channel to bundle average velocity was found to be 1.04 and **1.19** in the turbulent and laminar flow regimes respectively for **h/d =** 48. The corresponding values for the **6** in. lead run **(h/d =** 24) were 1.02 and 1.24 respectively. The theoretical flow split yields a value of 1.10 for  $\overline{V}_{wall}/\overline{V}_{bundle}$  for turbulent flow and 1.22 for laminar flow when no tolerances are taken into account. Whereas the theoretical flow split without tolerances appears to agree in the laminar flow case it yields a value approximatelv **6%** higher than the experimentallv measured value. When tolerances are taken into account a range is obtained for the flow split (see Section **3.1).**

Another interesting result is that  $C_1$  lies between 0.12 and 0.14 for  $h/d = 48$  and  $C_1 \approx 0.21$  for  $h/d = 24$ . The value of  $C_1$  for  $h/d = 48$  is close to  $C_1 = 0.12$  ( $h/d = 52$ ) determined from the ORNL data. It is necessary to point out that the swirl velocity was measured only at the midpoint between the rod and gap and  $C_1 = 0.13$  includes a correction for the assumed velocity profila which is applied to the

measured point velocity in order to determine an average **.4** velocity in the gap. An error analysis' shows that the approximate total error in measuring swirl velocity is between **11** and **15%.** The swirl velocity at the points **C,** B and **A** (in rod to wall gaps) were found to be very nearly equal for all wire-wrap locations. If the average subchannel swirl flow increased in going from **C** to **A** one would expect for all wire wrap orientations a higher swirl velocity at **A** than at **C.** This was not the case. This clearly shows that, contrary to the data in Ref. 12, the swirl flow is not a function of the subchannel position on the flat of the hexagonal duct wall. Although the magnitude of the swirl flow changes with different wire wrap positions no significant gradients in swirl flow along the duct wall were found at fixed axial positions.

The entrance length required to attain fully developed flow was found to lie between **6** and **8** in. This result aarees with a similar result obtained in Ref. **19.**

# **6.** Japanese **91** nin Salt Injection Tests (Ref. **13)**

The Japanese **91** pin alkaline (Fig. 12) salt injection tests were performed using a 0.248 in. diameter fuel pin and a **p/d** ratio of 1.20. Pitot tubes at the exit of the bundle measure the local velocity distribution in the subchannels. In addition dynamic and static deformation of the fuel rods produced **by** flow was measured **by** a strain gauge fixed on the surface of the fuel rod for the purpose of

knowing vibration and deformation of the rods. Pressure drop was obtained by measuring distribution of static pressure **by** a pressure gauge and obtaining pressure distribution at each water temperature and flow rate. For the mixing tests, sodium chloride solution was injected into water flowing through the fuel assemblies and concentration distribution of sodium chloride was measured at the exit end of the fuel assemblies. The detector probes are fixed to a probe fixing plate (Fig. 12) and placed at the center of several subchannels as shown in Fig. **13.** The subchannel velocity was measured with a standard deviation of **8.1%** and the standard deviation of flow measurement is 24.3%. The latter is higher because of axial bowing of rods which were determined from the vibration and deformation tests.

The mixing tests for the fuel assembly were performed with salt injection in central and noncentered interior subchannels. However only one single data point is available for central injection in the turbulent flow regime and no data points are available for turbulent flow for the eccentric salt injection.

Only the peak velocity profiles in each subchannel across the bundle have been reported. The peak velocity profile shows a W shaped distribution. The peak velocities near the wall are higher than their immediate neighbors and an increase in the peak velocity in the central regions is observed.

Relations obtained between the coefficient of friction { and Reynolds number Re from the pressure drop data show that they are in good aareement with the Blasius's equation for cylindrical tubes provided the representative lengths and flow velocity were calculated including the wire spacers.

### ENERGY **-** Analysis and Calibration:

One cannot infer the average velocity distribution from the reported peak subchannel velocity distributions. Consequently it is extremely difficult to determine a flow split from the reported peak velocity distribution. However a W shaped velocity distribution across a **91** pin bundle has also been reported earlier by Bump et al.<sup>14</sup>. If the average velocity distribution is in fact **W** shaped then non-uniform mixing coefficients might also be expected across the bundle. The degree of non-uniformity in transverse velocity distribution and its effect on mixing coefficient distribution is not yet resolved. Obviously if the non-uniformity in velocity distribution across the bundle is large, measurements of velocities in typical wall, corner and central channels is of no major gain. Unless the various codes take into account the "correct" velocity distribution the mixing coefficients obtained by calibrating the code with temperature data based on assumed velocity distribution would be erroneous. An outline of an experiment which would yield a variation of mixing coefficient across the bundle, if one exists, is given in Section **5.**

The ENERGY code was used to obtain  $\overline{\epsilon_{\text{H}}^{\star}}$  for the single set of data available for the fuel assembly in the turbulent flow regime. Fig. 14 shows the experimental data along with the best ENERGY fit. The  $\overline{\epsilon_H^*}$  so obtained was equal to 0.025 **+ 0.01** at a Reynolds number of about **7600.** It is interesting to note that Fig. 14 shows a slight asymmetry in salt concentrations about the injection point. For example, probes **3** and **3'** are located at the same radial distance from the injection point, yet probe **3'** records a lower concentration than the probe **3.** This asymmetry could be due to inlet perturbation effects or internal swirl or due to vibration and/or axial non-uniform deformation of fuel pins observed in the tests. ENERGY predicts a radially symmetrical distribution about the injection noint since there is no provision for taking into account internal swirl flow. In any case, the internal swirl flow appears to be very small. This has also been discussed earlier regarding the analysis of the ORNL data.

## **7.** Karlsruhe Heated Pin Experiments (Ref. **15, 16, 17)**

The Karlsruhe **61** pin experiments employ one or several electrically heated rods and sodium as coolant to determine mixing rates for several ratios of wire pitch to rod diameter. The rod diameter used in their study is **0.2362** in. which is very close to the FFTF geometry. However the **p/d** ratio is **1.315** and **h/d** ratios tested are **16.67,** 33.34 and **50.**

The test section consisting of the **61** rod bundle is inserted in a box which is conically supported in the sodium vessel of the test loop. The sodium enters a mixing chamber and flows downwards through the bundle, first through a **500 mm.** long hydraulic entrance region and then through the **1000** mm. long section with the heated rod. At the outlet of the bundle, **50** NiCr-Ni thermocouples of **0.5** mm. diameter (Fig. **15)** are installed in the different subchannels to register the radial temperature profiles in the bundle crosssection.

The experiments were made within the range **10 <** Re **5 < 10 .** The sodium temperature at the test section inlet was in the range **300 - 500\*C.** The tolerance on pin-to-pin spacing was less than **0.1** mm. Great care was taken to ensure any azimuthal variation of the heat flux due to fabrication tolerances in the heaters. In addition great care was taken in installing and in knowing the precise location of thermocouples within each subchannel. The thermocouple accuracy was not reported. We have assumed it to be  $+ 1^{\circ}F$ .

Data was taken, with the central rod heated, at two different Reynolds numbers for the three different **h/d** ratios. In addition Ref. **17** reports an experiment with two heated rods, one at the center and one at the wall for **h/d** ratio of **16.67.**

#### ENERGY **-** Analysis and Calibration:

Figs. **16** and **17** show two series of measured sodium temperature profiles. Plotted are the differences between local subchannel and sodium inlet temperatures as a function of the radial subchannel position for bundles with wire wrapped rods of different leads. In both runs the heating power of the center rod was nearly constant but the mass flow rate varied. It is seen that the sodium temperature profiles decrease very rapidly away from the central rod in the radial direction.

As the **h/d** ratio decreases mixing increases and the radial temperature gradient proportionality decreases. The measured temperature profiles were used to determine  $\overline{\epsilon_{\text{H}}^*}$ using the ENERGY code. The  $\overline{\epsilon_{H}^{*}}$  values were first determined from Fig. 16. The same values of  $\overline{\epsilon_{\rm H}^*}$  could predict the data well in Fig. **17** even though the Reynolds number was doubled to approx. **33000.** The values of  $\overline{\epsilon_{\text{H}}^*}$  determined from this experiment are:



The Karlsruhe experimenters do not report any asynmetry in radial temperature profiles. One may then assume that the effects of wire start position at entrance or internal swirls do not affect the exit temperature distribution. The swirl flow near the duct wall cannot be determined from the single heated pin experiment.

Ref. **16** and **17** report experiments with two heated rods. The temperature distribution from Ref. **16** is shown in Fig. **18.** Although an ENERGY analysis was not made because it is not specifically known which reported data points correspond to which subchannel, yet several interesting points are brought out **by** the experiments. The influence of swirl flow near the wall penetrates almost to the central regions of the bundle. This is seen from the difference in temperatures measured, at the same radial distance from center, along M **- C** and M **-** F in Fig. **18.** It is possible that part of the difference could be due to an internal swirl and one should not wholly attribute it to the penetrating effect of the wall swirl. However one cannot know which of these is predominant from a **61** pin experiment, where there are only 4 rows of rods from the wall to the center. Since the MISTPAL-II code<sup>16</sup> is reportedly able to predict the temperature distribution along MF and **MC** well althouah it does not have an internal swirl model in it, one is led to believe that the penetrating effect of wall swirl is dominating over any internal swirl that may exist.

Another interesting point is that MISTRAL-II uses uniform mixing coefficients in the bundle to match the data. This shows that the degree of interaction (in terms of mixing) between the inner and wall regions is not significantly smaller than the interaction between the subchannels within the inner core of the bundle. Although very near the wall (along MX) the temperature gradients are very small

(as found in Ref. 12), the steepness of temperature gradients existing in the radial direction near the wall is of the magnitude of the temperature gradients existing near the center of the bundle. This shows that the mixing coefficients are of the same order of magnitude in the radial direction except very close to the wall.

#### **8. ANL-RAS-** Hot Water Injection Tests (Ref. 12)

The **ANL-RAS** mixing tests were conducted with water in a 91-element bundle. The major emphasis is placed on investigation of flow characteristics in the peripheral channels. Hot water tracer was injected into selected wall channels and the axial and radial movement of tracer was detected with thermocouples mounted on the rod walls. Great care has been taken in developing and designing fast response thermocouples and the injection system. The injector design, its position and the injection flow rate have been optimized in order to get a consistent set of reliable results. It was found that the subchannel centered injector provides the best representation of the condition analyzed with codes using the rectangular edge subchannel geometry. Moreover with the centered injector there is greater probability of uniform mixing across the injection subchannel. Injection velocities slightly greater than nominal velocity through the bundle were found to give best mixing within the injection subchannel. Only one wire-wrap staring orientation was used in this experiment for obtaining data.

Fig. **19** shows the circumferential and axial progression of the dye and thermal tracer. It was found that the maximum concentration of the tracer lags behind the tracer front with the wire wrap angle almost midway between the two. Comparison of the thermal tracer results with the previous dye-tracer (Ref. **18)** results show excellent agreement. Both the locus of the maximum concentration and dye front show an in-phase periodic behavior. As the wire wrap moves from the adjacent interior subchannel into the edge subchannel the swirl flow is seen to increase.

The thermal-tracer studies of Ref. 12 also indicate that the swirl flow varies not only with wire wrap position as was found in Ref. **8,** but also with the position of the subchannel across the face of the bundle. The subchannels near the downstream corner of a face (in the direction of swirl flow) have the higher swirl flow. The authors of Ref. 12 believe that the corners are particularly important in the swirl flow mechanism. As the size of the bundle increases, they feel, "corner interaction" is less causing swirl flow to decrease. These experiments also show that the interaction between the edge subchannels and interior subchannels is low.

The ENERGY ANALYSIS and Calibration studies are given for subsections VIII and IX simultaneously at the end of subsection IX.

#### **9. ANL-CT-** Salt Injection Tests **(19)**

The **ANL-CT-91** pin tests were performed on a 2:1 bundle size. The rod dia. **p/d** and **h/d** ratios are **0.5** in., 1.24 and 48. The working fluid is water. The Reynolds number was varied from **9000** to 24000.

The main feature of this experiment is the isokinetic sampling technique. An electrolyte tracer is injected into the flowing water stream and isokinetically withdrawn at the exit of the bundle (see Refs. **6, 19** for details on isokinetic sampling technique) which permits one to measure both mixed mean subchannel concentrations and flow rates. Thus any errors in trying to determine subchannel average quantities from point measurements are avoided. Confidence in the accuracy of this technique in obtaining reliable concentration gradients in the first wire-wrap pitch of an assembly was first obtained on a 7-pin bundle.<sup>6</sup>

Considerable effort has been devoted to optimize the injector design and flow rates so as to get accurate measurements of concentrations downstream of the injection point. In addition care is taken to ensure that the initial wire orientation remain fixed with respect to the injection position.

The flow split (  $V$ channel/ $\overline{V}$  ) obtained from these experiments **by** isokinetically sampling the flow show different results than the predictions based on hydraulic diameter concept. The subchannel to the bundle average velocity ratio for the central, edge and corner subchannels was found to be **1.01, 0.99** and 0.94

respectively. Time-of-flight measurements on the RAS-91-pin bundle<sup>12</sup> using a thermal pulse and the AI 217 pin bundle<sup>20</sup> using a salt tracer pulse yielded edge channel velocity split to be **1.03** and **1.00** approximately.

There was no Reynolds number effect on the concentration data or on the flow split data. In order to assess how long it takes for the flow to fully develop two extreme entrance conditions were employed; on a **70%** circular plan blockage of the central region of the bundle and the other a **50%** annular planar blockage of the peripheral regions. The isokinetic velocity measurements show identical results at 2 1/2 spacer pitches from the entrance. Obviously any initial flow maldistributions have died away. Similar results were obtained for blockages near the exit.

Fig. 20 shows the trace of the maximum concentration and the dye front for wire wrap orientation **A.** Fig. 21 shows the same for wire wrap orientation B at the injection point. It is seen that the average slopes of both sets of data and similar data from thermal-tracer experiments (Fig. **19)** are very close to each other. In addition for the same wirewrap orientation the trace of the maximum concentration and the dye front are also in phase for the two different experiments (Ref. 12 and **19).** These similarities in data exist in spite of the diameter and wire wrap pitch in Ref. **19** being twice that of Ref. 12 showing **h/d** is a correlating parameter.

### ENERGY **-** Analysis and Calibration

**A** pulse-flow model was incorporated in ENERGY to take into account an enhanced translation in movement of the wavefront. **As** shown in Fig. 22 due to smearing effect in lumped parameter codes, the calculated wave front is always located ahead of the actual wavefront; the difference between the two depends upon the mesh size. Although the trace of the maximum concentration is not affected **by** smearing to any appreciable extent the predicted tracer front is affected. Fig. **23** shows that the pulse flow model results match the tracer maximum concentration as well as the dye front for  $C_1 = 0.14$ . The tracer front was represented **by** lines of constant concentration between **0.005** and **0.05.** It is necessary to note that the predicted wavefront follows the experimental wavefront closely from inlet to exit.

Fig. 24 shows comparison between the data and the ENERGY model without the pulse flow model in the wall region. It was again found that  $C_1 = 0.14$  is able to predict the trace of maximum concentration well. However, as discussed earlier, the tracer front initially travels ahead of the experimental tracer front due to the "smearing-effect." It is necessary to point out that we believe that the bulk of the tracer injected must travel in the direction of the velocity vector. Thus the trace of the maximum concentration represents the velocity vector and the tracer front is ahead of all the

maximum concentration line due to turbulent diffusion effects. **10.** GE-127 pin Hot Water Injection Tests (Ref. 22)

The **127** pin bundle was **72** in. long and contained a hexagonal flow duct. The diameter of the pins is **0.25** in., **p/d** and **h/d** are **1.28** and 24 respectively. The Reynolds number of the test was about **55000.** Hot water is injected and mixing rates were inferred from temperature measurements downstream. Fig. **25** shows the main test loop. The loop temperature was maintained at 100°F. Approximately 2% of the main flow was diverted from the loop, just upstream from the main pump, heated in a 900,000 Btu/h boiler to 161.4<sup>O</sup>F, increased in pressure up to **125** psi with a small 2 **hp** jet pump and reinjected into the center of the bundle. Fig. **26** shows the cross-section of dummy fuel bundle and Fig. **27** shows the mechanical details of hot water injection. Lateral dispersion of hot water was inhibited for a length of one inch above the injection tube **by** a "dam" in the form of a filler rod welded to each of six rods surrounding the central injection tube. The flow rate in the injection flow channel is approximately made equal to the flow rates in the **6** central channels. Care was exercised in the adjustment of injection and main flow rates during experimentation so as to minimize any localized lateral dispersion due to mismatch in flow velocities.

Lattice temperature measurements downstream from the hot water injection point were measured at every **1** in. axial distance from **6** in. to **65** in. Fig. **28** shows the lattice temperature probe assembly. This probe assembly was designed to fit within the tube which simulated fuel rods and could measure the temperature of the moving fluid in any one of the six channels surrounding the instrumented rod without perturbing the flow. The probe assembly could be moved along the **72** in. length of the dummy fuel rod and could be rotated within any particular tube so as to orient the thermocouples preferentially in any direction. In order to confirm the directional temperature sensitivity of the probe assembly two dimensional heat transfer calculations were done on the probe assembly, dummy fuel rod and water environment for several assumbed circumferential temperature distributions. one severe distribution is shown in Fig. **29.** In this case the thermocouple response was calculated to be **950F** showing considerable directional sensitivity.

It is obvious from Fig. **26** that estimation of swirl flow in the wall regions cannot be made from this setup. Therefore the effort was concentrated on determining  $\overline{\epsilon_{\text{H}}^{\star}}$  for the particular geometry and flow conditions.

In order to determine  $\widetilde{\epsilon_{\rm H}^*}$  for salt and hot water injection experiments, the following procedure was adopted. The temperature rise for various channels is determined from the ENERGY code by varying the input parameter  $\overline{\epsilon_{\rm H}^*}$ . Fig. 30 shows a plot of temperature rise vs.  $\overline{\epsilon_{\text{H}}^{\star}}$  for several channels, at the 12 in. axial level and at the **36** in. level. On the same figure the

recorded data is plotted showing a  $\pm$  1<sup>O</sup>F spread due to assumed thermocouple accuracy. It is interesting to note that the slope of the curve in Fig. **30** for the injection channel at 12 in. is large but for other channels the slope is not large at 12 in. At **36** in. the slope of all the curves are small. The only meaningful estimate of the range of  $\overline{\epsilon_{\rm H}^{\star}}$  can be obtained from the injection channel data at 12 in. For other channels, when  $a + 1^0F$  spread on data is superimposed on the plotted computer curves, it is seen that almost any value of  $\overline{\epsilon_{\text{H}}^*}$  should predict a temperature rise within the data range. Thus usefulness of the data **6** to 14 in. beyond injection is limited. The value of  $\overline{\epsilon_{\text{H}}^*}$  so determined as found to lie between **0.050** and **0.060.** The procedure described above is only the first step in data analysis. The next step, a refinement on Step I, is described below.

Since at the injection point there is a perturbation of the main flow stream due to injection of salt or hot water, it is quite conceivable that these perturbation effects may last several inches downstream of injection point. The data is then analyzed **by** taking the inlet plane in the computer program to be several inches downstream of the injection point in the experiment. For this data inlet was assumed to be 12 in. downstream of injection point and data at this 12 in. location was assumed as the initial channel condition. Fig. **31** shows the data for the injection channel and also the predictions of ENERGY for **5** different cases. Set I shows the predictions

for the case where the inlet to ENERGY is at the same plane as the injection point. For other curves the 12 in. level in the experiment is the starting point for the calculations. It is seen that curves I and II do not match although  $\overline{\epsilon_H^*}$  = **0.062** for both **.** This was found to be due to a lack of energy balance in the data between inlet and 12 in. level. The total energy at the 12 in. level, as calculated **by** taking the sum of the product of flow and enthalpy over all channels, was found to be greater than the total energy in the injection plane. The data at 12 in. is probably more reliable **(** as discussed below **)** and free of any inlet perturbation effects. The inlet energy balance was corrected to match that at the 12 in. level. On superimposing  $a + 1^{\text{OF}}$  error band on the data  $\widetilde{\epsilon_{\rm H}^*}$  was found to be in the range 0.040 to 0.070 or an approximate average of **0.055 + 0.015.**

The data after the 12 in. axial level is probably more reliable because of the following reason. It was found that the radial temperature distribution across the bundle had a peak which rotated circumferentially as the fluid travelled up the bundle. Perhaps an internal swirl had generated due to slightly asymmetric injection into the six inner channels. This internal swirl appears to die off at about 12 **- 18** in. when the radially peak temperature stops rotating. Thus as one proceeds axially downstream the reliability of the data increases but its sensitivity to  $\overline{\epsilon_{\rm H}^*}$  decreased (Fig. 30). Therefore the 12 in. level data was chosen as a reliable

set of data to start calculations and yet maintain reasonable sensitivity to  $\overline{\epsilon_{\text{H}}^*}$  variations.

### **11. HEDL-217** PIN **SALT** INJECTION **TESTS**

The **HEDL 217** pin tests are the only set of experiments on a full size FFTF fuel assembly geometry. Considerable effort, therefore, has been devoted to developing an understanding of the experimental set up instrumentation, test procedure and results.

Three types of pins were used for the assembly **(1)** Dummy pins, (2) Instrumented pins, and **(3)** injection pins. The **HEDL** description of their sensors is as follows: "Fifty-eight instrumented pins were fabricated from laminated phenolic tubing, **NEMA** grade XXX. Three basic electrode arrangements were necessary to provide the required measurement patterns. The fabrication of the pins was similar for each of the arrangements. First, the 1/4 inch **OD** x **3/15** in. ID phenolic tubing wall was drilled with the required electrode pattern, then a length of **17** guage B **& S** Forms air insulated copper wire (0.045 in. dia.) was pulled through the tube and out each hole. Each pin had eight electrodes. The tube bore was filled with epoxy resin to provide an insulating seal. Each pin was then centerless ground to an outside diameter of **0.23** in. and the exposed copper electrodes nickel plated. The pins were trimmed to length and a lower end cap which engages the lower grid, was attached with epoxy glue. The

end cap slot was oriented as required to assure the proper electrode location and wire wrap phase."

**A** salt solution can be injected into the subchannels from three injection pins marked **1,** 2, **3** in Fig. **32.** An equal amount of fluid is withdrawn in order to minimize any disturbance of the main fluid stream. In order to insure that the injected salt solution completely mixed within the subchannel with the mainstream fluid three fins were provided which isolate the subchannel up to 2 3/4 in. from the salt injection point. The addition of surface area due to the fins was compensated **by** a reduction in wire-wrap surface area so that there is no axial reduction in flow in the injected subchannel. Fig. **32** shows the three injection rods, wall injection location and instrumented rod locations. **A** few subchannels were surrounded **by** two instrumented pins.

**As** indicated **by** Ref. **23,** not much effort was devoted to an orderly instrumentation development including injector design and its effect on results. However, Ref. 12 states based on their experience that the injection effect on results is expected to be small.

The mass balance obtained show a **13%** error for interior injections and up to **37%** error for wall injections.

The sensitivity of the electrodes to electrolyte concentration was localized to the volume of the fluid immediately adjacent to the exposed electrode surface. Thus point measurements were obtained in these tests as opposed to subchannel

average measurements obtained **by** isokinetic sampling techniques. Thus if high concentration gradients exist within a channel the reading would have been weighted toward the concentration closest to the probe. This effect is termed striping. The authors of the **HEDL** report feel that near the injection point and a few inches downstream high intrachannel concentration gradients existed and probe readings were not the bulk concentration of the subchannels. However, downstream where concentrations in adjacent channels were similar, the striping effect would be negligible.

Typical results for central injection are shown in Fig. **33** (a,b,c,d). The injection concentration was **2165** ppm (the mass balance shows **1991** ppm at **6** in. axial level, Fig. 33a). At 12 in. the concentration in the injection channel has dropped down to 194 ppm, the mass balance shows **2206** ppm. At the **36** in. level the concentration in the injection channel is down to **63** ppm but the mass balance is further up to **2370** ppm. In addition to the mass balance being slightly off there was an apparent drift (which could be due to a net mass transport in the direction of the wire twist or due to inlet perturbation effect) in the direction of the wire rotation.

Typical results for wall injection are shown in Fig. 34 a,b,cd. It is seen that there is considerable interaction between the wall and the first row of interior subchannels. In addition one finds that in going from zero to an axial level of **36** in. the maxima in concentration moves only (4-5) subchannels along the circumference of the duct while the tracer front advances only 2 subchannels ahead of the first corner channel in its path. The mass balances for wall injection were poor as the probes were damaged either from vibration or from swelling of the epoxy due to exposure to water. (Ref. 12)

In none of the tests performed did the authors (of Ref. **23)** find any dependence of their results on Reynolds number in the range **10000 <** Re **< 30000.**

#### ENERGY **-** Analysis and Calibration:

The two major issues, apart from ENERGY calibration, that have to be resolved **by** an ENERGY analysis were as follows. First, whether the swirl flow, measured by  $C_1$  in ENERGY is lower than found for the ORNL and **ANL** bundles of similar geometry. If it is lower then is it due to improper response of instrumentation or is it a bundle size effect. Second, is the drift observed for internal injection caused **by** internal swirl or is it caused **by** an inlet perturbation effect?

Table IV gives the **HEDL** runs analyzed. Runs 4, **11,** 14 and **19** were analyzed for interior injection and Run 2 for wall injections. The method of analysis of this data was similar to that described earlier in subsection X for the **GE-127** pin data analysis. Initially the injection plane of the tracer was used as the starting level for code calculations. It was found that the exit concentrations at the 36 in. level could not be matched for any value of  $\overline{\epsilon_{\text{H}}^*}$ and **C1 .** Next the 12 in. level **HEDL** data was used as the starting plane for code calculations.

Every set of data analyzed could be predicted well at the **36** in. level if calculations were started at the 12 in. level of the **HEDL** data. **A** typical ENERGY comparison with data is shown in Fig. 35. The  $\overline{\epsilon_{\text{H}}^*}$  found (Table 4) lies in the range  $0.015$  to  $0.030$ . The range for  $\overline{\epsilon_{\rm H}^*}$  so found is not large considering the uncertainties in mass balance. However, one important conclusion can be readily drawn. Since ENERGY using the 12 in. plane as initial condition can predict the data one can conclude that there is no internal swirl in the bundle. If internal swirl exists its effect on exit salt concentration is negligibly small. The so-called "drift" of two channels to the left of injection channel is an injection perturbation effect. When the starting level for the calculations was taken at the **6** in. level of the data the exit **(36** in)concentrations could not be predicted as well. This shows that the inlet perturbation effects probably persisted up to 9 in. from the injection plane. The  $\overline{e_H^*}$  found from **HEDL** tests is considerably lower (30-40%) than that found from the ORNL heated pin test. In order to explain this difference the data was examined closely. As seen in Fig. **33** some of the channels had two probes in them. The probes pointing in a direction opposite to wire map rotation (wire wrap rotation and swirl flow are clockwise for **HEDL** tests) always measured higher concentrations than the other probe in the channel. Often the difference in the concentration measured **by** the two probes was large. If other channels had
been equipped with more than one probe it would have been easier  $t \epsilon$  estimate the average concentration in the channel. Obviously point measurements are difficult to use in calibrating lumped parameter codes. Another reason for obtaining low  $\overline{\epsilon_{\text{u}}^*}$  in these tests is the possibility of change in accuracy of data with salt concentration.  $24$  If the salt concentration is increased to a high value, the electrical resistance that is measured by conductivity cells is low and accuracy is small.<sup>24</sup> If the salt concentration is very low the accuracy with which concentrations can be measured is high and the concentration gradients might be low within a subchannel. However, very small errors in trying to relate point measurements to bulk subchannel values could give a wide spread in the  $\overline{\epsilon}_{H}^{*}$  values. An intermediate concentration range is desirable. However the wall probes in the **HEDL** data are subject to (laminar) boundary layer effects which can increase electrical resistance many times leading to a loss in accuracy.<sup>24</sup>

Figs. 36a and **b** show the rate at which the maximum in subchannel concentration travels along the duct wall. The slope of the trace of the maxima is considerably smaller than found from the **ANL** data (Figs. **19,** 20). Since the maxima in concentration represents the swirl velocity more closely than does the tracer front it is obvious that the swirl velocity is smaller for this **217** pin bundle than for the **91** pin bundle. Fig. 37 shows that for the HEDL bundle  $C_1 \approx 0.07$ , which could also be determined from Figs. 36a and **36b.**

Contrary to the observations in the **ANL** (Ref. 12) considerable interaction with the interior channel took place.

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The reasons for obtaining a lower C<sub>1</sub> for these tests have been discussed in the main text. Either the point probes located in the rod walls do not accurately measure salt concentration in the wall channels or there is an actual bundle size effect.

### **7.2** Definition of Mixing Mechanisms

Energy redistribution in a wire-wrapped fuel assembly takes place **by** the following mechanisms:

- (a) Thermal Conduction **-** characterized **by** the thermal diffusivity, a.
- **(b)** Turbulent Exchange on a Molecular Level (including flow scattering) without a net transfer of fluid. The dimensionless group characterizing turbulent exchange is,  $W_{\text{I}_1}$  / $\overline{W}$ , where  $W_{\text{I}_1}$  is the turbulent exchange rate and  $\overline{W}$  is the average bundle flow rate.
- (c) Cross-Flow **-** any convection of fluid due to a radial pressure gradient can be classified as cross-flow. Cross-flow can be subdivided into two categories: **(1)** Diversion Cross-Flow: That fraction of the total cross-flow between any two sub-channels that occurs due to a pressure gradient set up **by** virtue of the dissimilarities in local hydraulic characteristics (either geometrically, hydrodynamically or thermally induced). For example, (figure **1)** as the wire lead to a diameter h increases to very large values ( $\overline{d}$  <del>+</del>  $\infty$ ) the hydraulic diameter of channels i and **j** would be locally different at various axial levels causing flow to redistribute itself **by** diversion cross-flow. Thus the varying axial and transverse flow resistance in the

presence or absence of wire wrap can cause diversion cross**flow.**

Diversion cross-flow may be characterized **by** the term,

 $W_{\text{Di}}$ <sup> $\sqrt{W}$ </sup>

(2) Flow-Sweep: For wire-wrapped rod bundles of **p/d > 1.18** the axial flow has no direct vertical path through the bundle **(1)** but at regular intervals crosses the wire and is swept from one channel to another due to the favorable pressure gradient set up **by** the wire. This sweeping effect of the wire would extend a small distance below and above the location where the wire crosses the gap. Thus the axial momentum of flow in a subchannel is periodically changed **by** the presence of the wire. The fraction of axial momentum carried **by** the sweep flow in the transverse direction can be considerably greater than that carried **by** diversion cross-flow, depending upon the wire lead to diameter ratio. For rod bundles with **p/d < 1.18** a part of the fluid can flow vertically upwards in a bundle and part of it will be periodically swept into other channels. The flow field for such a bundle would be even more complex than for a bundle with **p/d > 1.18.**

Flow sweep may be characterized by the term,  $W_{\text{S}i\text{i}}/\overline{W}$ (or  $W_{\text{S}i} / W_i$ ).

Swirl Flow: The sweep flow in the wall channels has characteristics which are different from that in the central channels. Whereas the sweep flow between two subchannels in the central parts of the bundle changes direction periodically, the sweep flow between two wall channels is always in the same

direction. This sweep **flow** along the housing wall is known as the swirl flow. The swirl flow, perhaps, fluctuates about a mean value. It is characterized by the term,  $V_{\theta}/\overline{V}$ , where V<sub>e</sub> is the velocity in the gap between the rod and the wall and  $\overline{V}$  average axial bundle velocity.

Most experimental effort has been to quantitatively determine these four mechanisms individually if possible and their dependence upon geometrical and flow parameters. The major emphasis has been in determining flow sweeping, perhaps because it is the least understood and the most important means of momentum and energy transfer. Once these mechanisms are known, they are direct input into the subchannel analysis computer programs as they are now formulated.

# Appendix

# Titles for Illustrations

# Figure No.

 $\mathbf{r}$ 





 $\label{eq:2.1} \frac{1}{\sqrt{2}}\int_{0}^{\infty}\frac{1}{\sqrt{2\pi}}\left(\frac{1}{\sqrt{2\pi}}\right)^{2\alpha} \frac{1}{\sqrt{2\pi}}\int_{0}^{\infty}\frac{1}{\sqrt{2\pi}}\left(\frac{1}{\sqrt{2\pi}}\right)^{\alpha} \frac{1}{\sqrt{2\pi}}\frac{1}{\sqrt{2\pi}}\int_{0}^{\infty}\frac{1}{\sqrt{2\pi}}\frac{1}{\sqrt{2\pi}}\frac{1}{\sqrt{2\pi}}\frac{1}{\sqrt{2\pi}}\frac{1}{\sqrt{2\pi}}\frac{1}{\sqrt{2\pi}}$ 

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 $\label{eq:2.1} \frac{1}{\sqrt{2}}\int_{0}^{\infty}\frac{1}{\sqrt{2\pi}}\left(\frac{1}{\sqrt{2\pi}}\right)^{2\alpha} \frac{1}{\sqrt{2\pi}}\int_{0}^{\infty}\frac{1}{\sqrt{2\pi}}\left(\frac{1}{\sqrt{2\pi}}\right)^{\alpha} \frac{1}{\sqrt{2\pi}}\frac{1}{\sqrt{2\pi}}\int_{0}^{\infty}\frac{1}{\sqrt{2\pi}}\frac{1}{\sqrt{2\pi}}\frac{1}{\sqrt{2\pi}}\frac{1}{\sqrt{2\pi}}\frac{1}{\sqrt{2\pi}}\frac{1}{\sqrt{2\pi}}$ 

 $\label{eq:2.1} \frac{1}{\sqrt{2\pi}}\int_{\mathbb{R}^3}\frac{1}{\sqrt{2\pi}}\int_{\mathbb{R}^3}\frac{1}{\sqrt{2\pi}}\int_{\mathbb{R}^3}\frac{1}{\sqrt{2\pi}}\int_{\mathbb{R}^3}\frac{1}{\sqrt{2\pi}}\int_{\mathbb{R}^3}\frac{1}{\sqrt{2\pi}}\int_{\mathbb{R}^3}\frac{1}{\sqrt{2\pi}}\int_{\mathbb{R}^3}\frac{1}{\sqrt{2\pi}}\int_{\mathbb{R}^3}\frac{1}{\sqrt{2\pi}}\int_{\mathbb{R}^3}\frac{1$ 

 $\label{eq:2.1} \begin{array}{c} \mathcal{F}^{\mathcal{C}}_{\mathcal{C}}(x,y) = \mathcal{F}^{\mathcal{C}}_{\mathcal{C}}(x,y) \, , \\ \mathcal{F}^{\mathcal{C}}_{\mathcal{C}}(x,y) = \mathcal{F}^{\mathcal{C}}_{\mathcal{C}}(x,y) \, , \end{array}$ 

 $\label{eq:2.1} \frac{1}{\sqrt{2}}\left(\frac{1}{\sqrt{2}}\right)^{2} \left(\frac{1}{\sqrt{2}}\right)^{2} \left(\$ 



TABLE **I**

**Z**TYPE OF EXPERIMENT (a) HOT WATER **INJECTION (b) SALT INJECTION**

(c) **HEATED PIN (d)** LASER



 $\mathbf{A}^{\mathcal{A}}$ 



 $\Box$ 

 $\mathcal{A}^{\mathcal{A}}$ 

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\* **NUMBERS** REFER TO TABLE - I

 $\mathbf{v}$ 

 $\sim 10^{11}$  km



 $\bar{t}$ 

 $\frac{1}{\lambda}$ 

Table III: SKOK-Bundles **&** Energy Coefficients

 $\mathcal{L}$ 



 $\label{eq:3.1} \frac{1}{2} \int_{0}^{2\pi} \frac{1}{2\pi} \left( \frac{1}{2\pi} \right)^{2} \frac{1}{2\pi} \int_{0}^{2\pi} \frac{1}{2\pi} \left( \frac{1}{2\pi} \right)^{2} \frac{1}{2\pi} \int_{0}^{2\pi} \frac{1}{2\pi} \frac{1}{2\pi} \frac{1}{2\pi} \int_{0}^{2\pi} \frac{1}{2\pi} \frac{1}{2\pi} \frac{1}{2\pi} \frac{1}{2\pi} \int_{0}^{2\pi} \frac{1}{2\pi} \frac{1}{2\pi} \$ 

 $\label{eq:2.1} \frac{1}{\sqrt{2}}\int_{\mathbb{R}^3}\frac{1}{\sqrt{2}}\left(\frac{1}{\sqrt{2}}\right)^2\frac{1}{\sqrt{2}}\left(\frac{1}{\sqrt{2}}\right)^2\frac{1}{\sqrt{2}}\left(\frac{1}{\sqrt{2}}\right)^2.$ 



 $\mathbb{R}^{n \times n}$ 





 $\lambda_{\rm{max}}$ 







 $\frac{1}{2}$ 

**FIG.** 4

 $\frac{1}{2}$  .

### **x LOCATION** OF EXIT THERMOCOUPLE



**TEST** PARAMETERS: REF **9**

- **1. FLUID - LIQUID** Na
- 2. FLOW RATE **- UP** TO **56 GPM** (Re **- I,000** TO **70,000)**
- **3.** POWER SKEW
	- **a. ALL** RODS UNIFORMLY **HEATED**
		- **b. EACH** ROD **HEATED INDIVIDUALLY**
	- c. B 20 **%** HIGHER **THAN E**
	- **d. E** 20 **%** HIGHER **THAN** B
	- e. B **300%** HIGHER **THAN E**
	- **f. E** 300%HIGHER **THAN** B
- 4. **d = 0.23 IN. h/d =52**
	- **p/d =** 1.24



FIG. 6





**FIG. 8**















**JAERI-memo 4577**



FIG. 12





O THERMOCOUPLE LOCATION





FIG. 14









FIG. 17



 $\sum_{i=1}^{n}$ 

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FIG. 18









 $\overline{\phantom{a}}$ FIG. 21



 $\mathcal{L}_{\mathcal{A}}$ 

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 $\sim$   $\sim$ 

FIG. 22






FIG. 24



FIG. **25**



FIG. **26**



FIG. **27**

 $\mathcal{A}^{\mathcal{A}}$ 

 $\sim 10^7$ 



**FIG. 28**

 $\mathcal{L}^{\pm}$ 



**CALCULATED** THERMOCOUPLE **RESPONSE (95\*F)**

FIG. **29**



FIG. 30





12 IN. BELOW FINNED REGION

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 $\hat{\mathcal{A}}$ 

 $\mathbb{C}$ 

أفعلوه والمتمرد

 $\mathcal{A}$  is the contraction of the space of the space  $\mathcal{A}$  ,  $\mathcal{A}$ 

 $\sim 100$  km

 $\sim 10^{-10}$ 

 $\sim 10^7$ 

 $\frac{1}{2}$  ,  $\frac{1}{2}$ 



 $\mathcal{A}^{\mathcal{A}}$ 

 $\sim 10^{-1}$  and  $\sim$ 

A-112

 $\mathcal{L}_{\mathcal{A}}$ 







 $\sim 10^7$ 

 $\sim 100$ 



 $\sim 10^{-10}$  km s  $^{-1}$ 





 $\mathcal{L}(\mathcal{$ 











*4-*





 $\sim 10^{-1}$ 

 $\bullet$ 

 $\bar{\Delta}$ 





GRAB SAMPLE - 102

 $\sim$ 













 $\sim 100$ 



**FIG. 35**

 $\mathbf{r}$ 



FIG. 36a

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 $\hat{\mathbf{r}}$ 

**FIG. 36b**

 $\sim$ 



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